



НИИАР

EP-450 Steel as Cladding Material for Fast Neutron Reactor Fuel Rods

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JSC “SSC – RIAR”

Material	Composition, % mass.								
	C	Si	Mn	Cr	Ni	Mo	Nb	V	B
EP-450	0.10-0.15	≤ 0.5	≤ 1.0	11.0-13.5	0.05-0.30	1.5-2.0	0.15-0.40	0.1-0.3	0.005-0.015

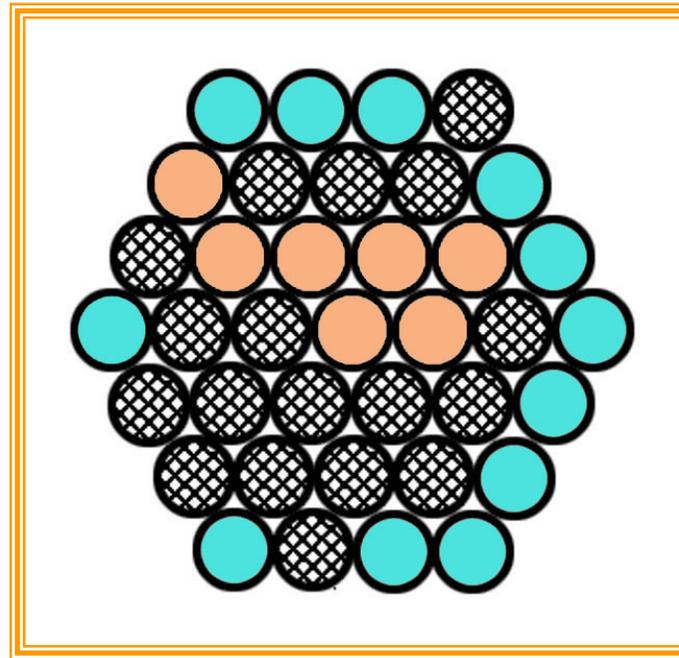
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Main irradiation parameters of fuel rods with EP-450 steel claddings and pelleted oxide fuel in BOR-60 and BN-350 reactors

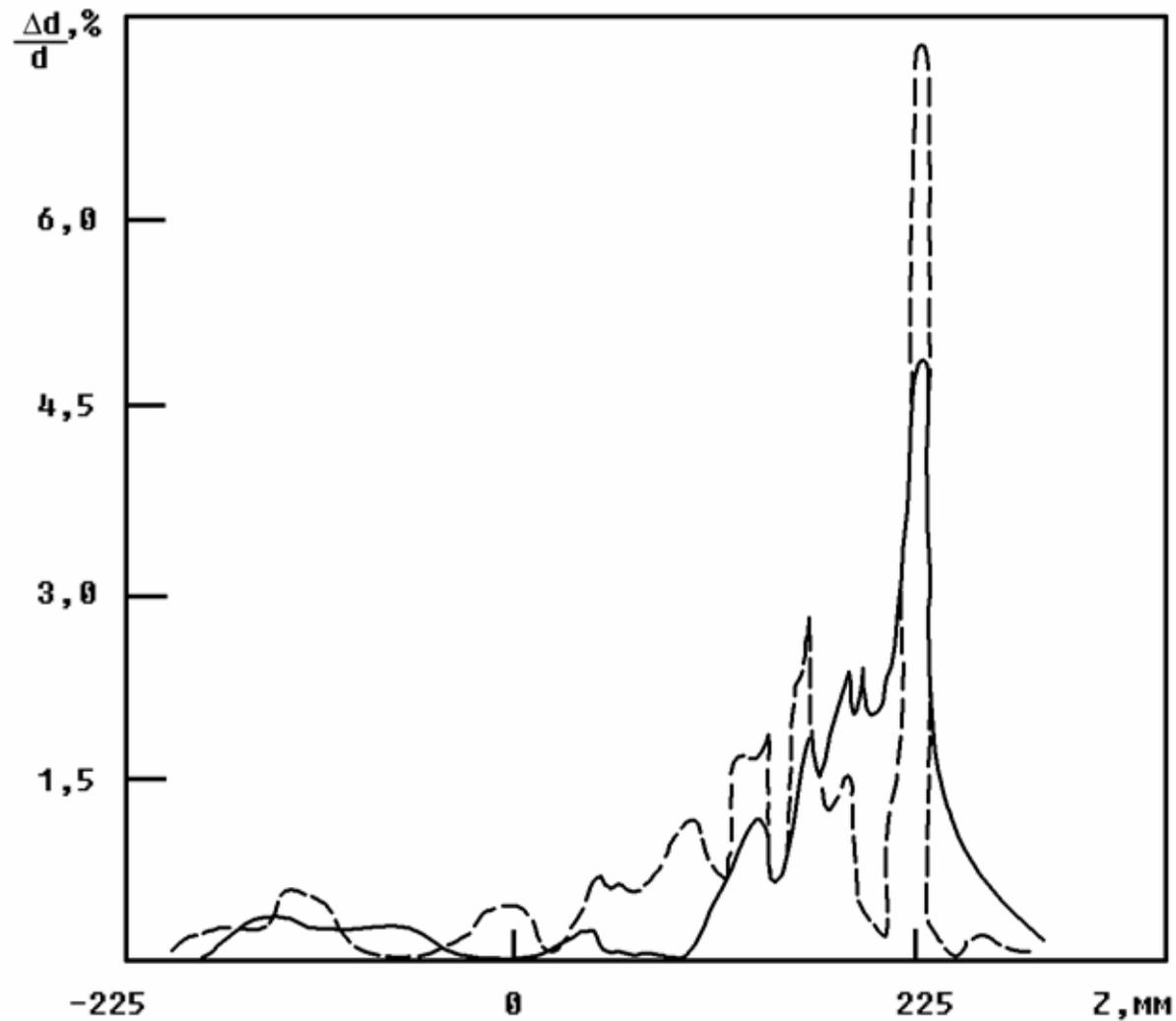
Reactor	FA index	UO ₂ fuel pellets	Burnup %, h.a.	Dose, dpa	Test time at power, h	Linear power, W/cm	Temperature of cladding internal layer, °C	Test time within temperature range, h
BOR-60	A-336	90% enrichment U ²³⁵	12,3	49	10420	> 477 477-400	> 667 667-620	1398 8685
	A-337	- « -	13,4	41,5	13840	> 464 464-340	> 690 690-630	2970 9903
	OP-21	- « -	10,8	42	10080	> 490 490-400	725-700 700-655	1090 5780
	OP-22	- « -	10,4	40	- « -	- « -	- « -	- « -
	OP-23	- « -	10,8	42	- « -	- « -	- « -	- « -
	OP-24	- « -	10,6	41	- « -	- « -	- « -	- « -
	OP-25	- « -	12,5	45	12500	450	>730 730-700	1640 8500
BN-350	OP-5	32,8 % enrichment U ²³⁵	10,6	48	7500	536-430	690-587	-
	OP-6	33% enrichment U ²³⁵	11,2	60	9750	470-360	650-537	-

Chart of the OP-25 FA

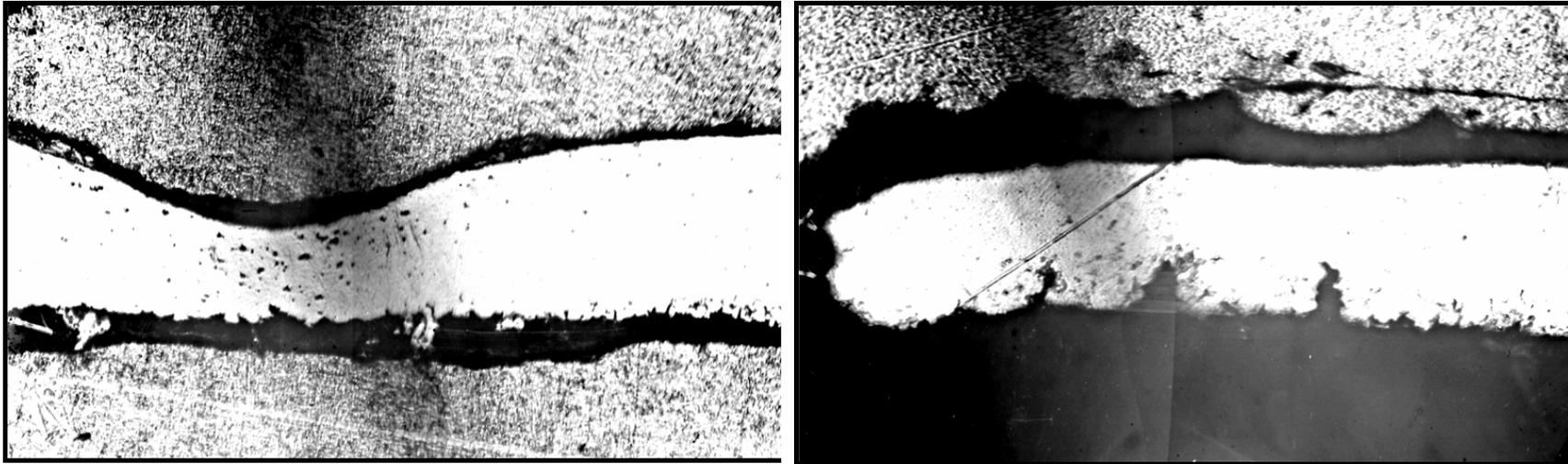


-  - tight fuel rods;
-  - leaky fuel rods;
-  - damage fuel rods .

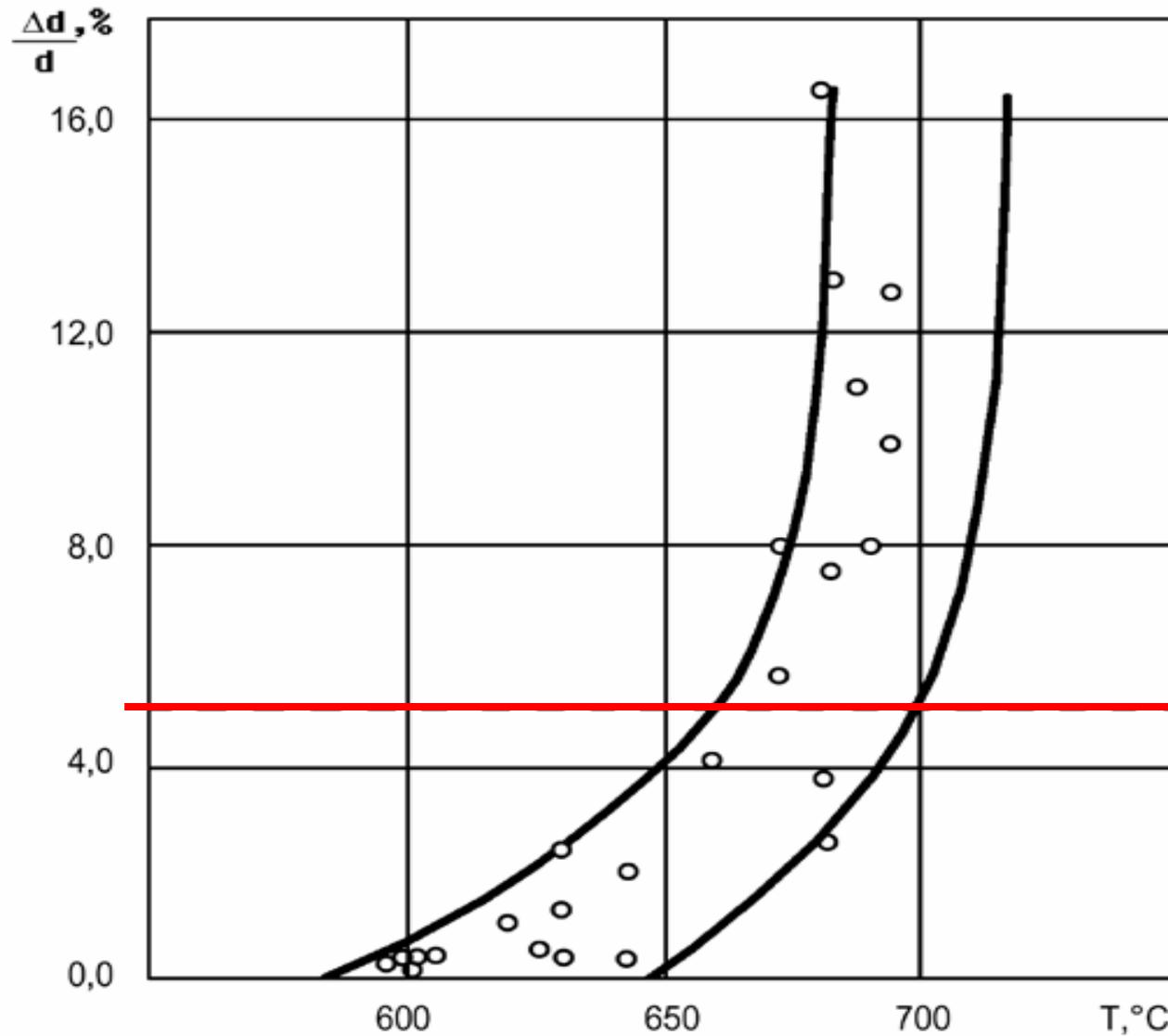
Typical diametral strain in OP-25 fuel rod



Damage fuel rods

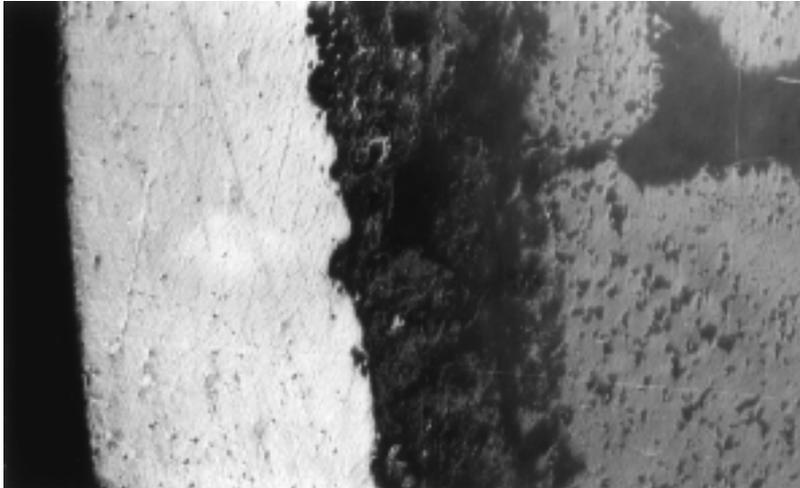


Typical areas of local thinning damage areas and several non-through ruptures of the cladding in the upper part of the fuel rods

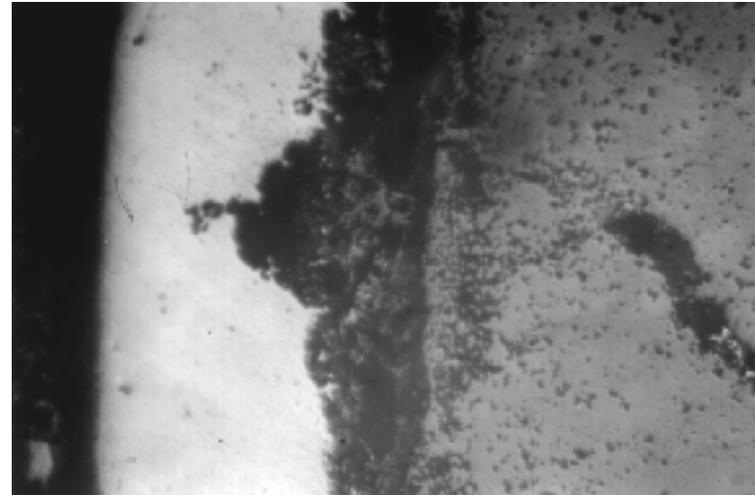


Maximum diametral strain in OP-25 fuel rods (BOR-60 reactor) vs. maximum initial irradiation temperature (--- level of maximum allowable form change)

Corrosion



a

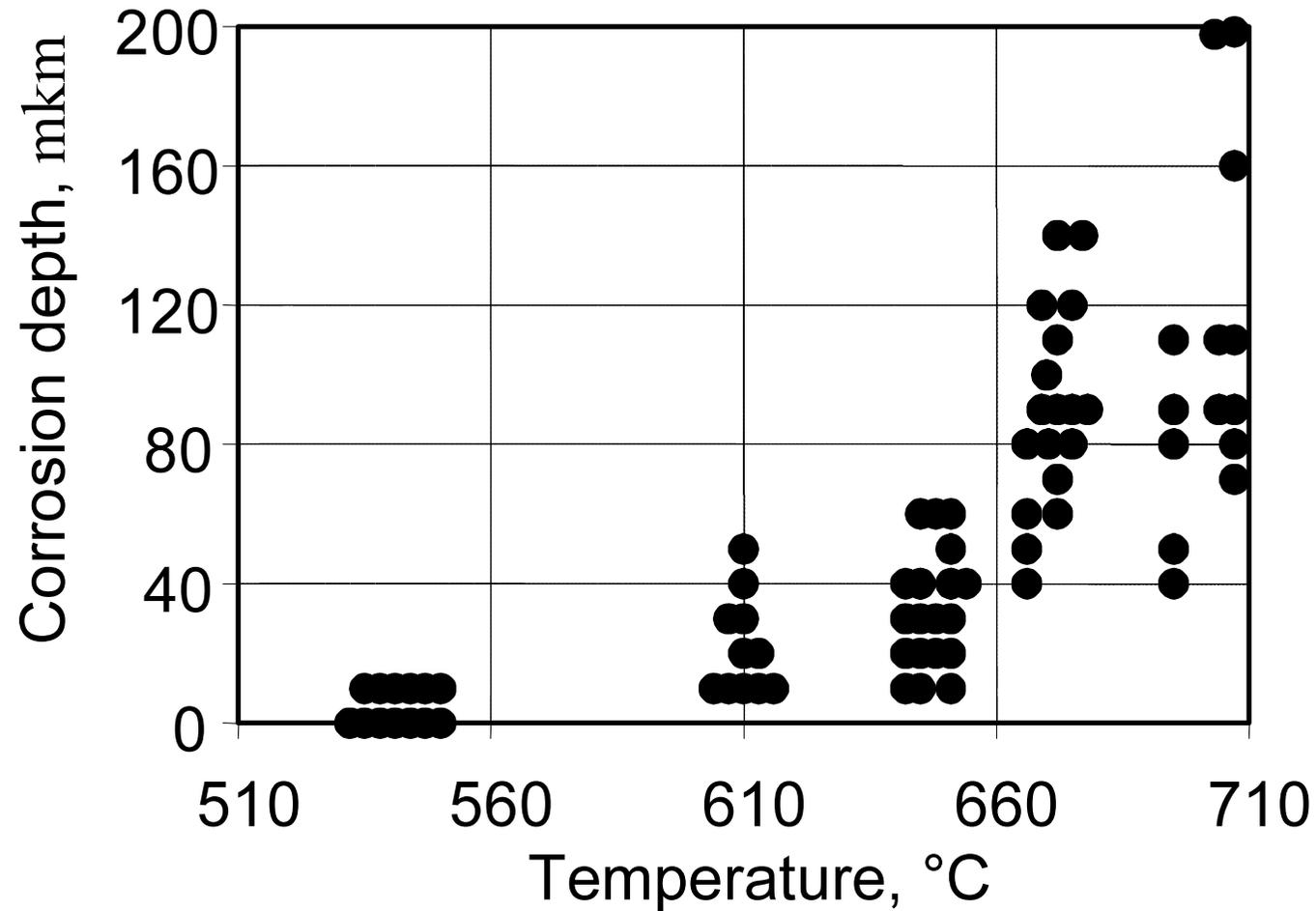


b

Structure of cross section fragments of OP-21 fuel rods after irradiation:

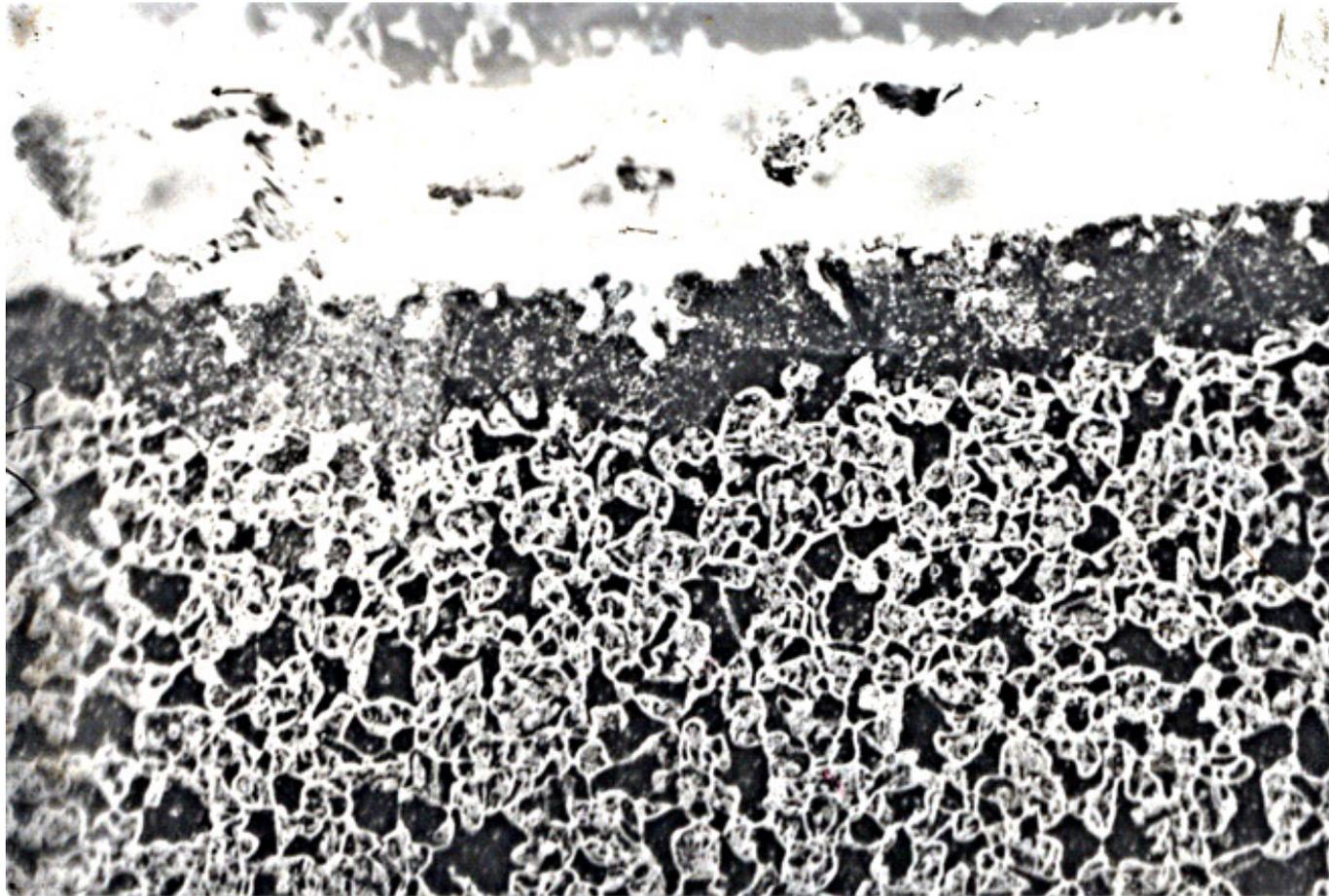
- a) cladding matrix corrosion;**
- b) cladding pitting**

Corrosion



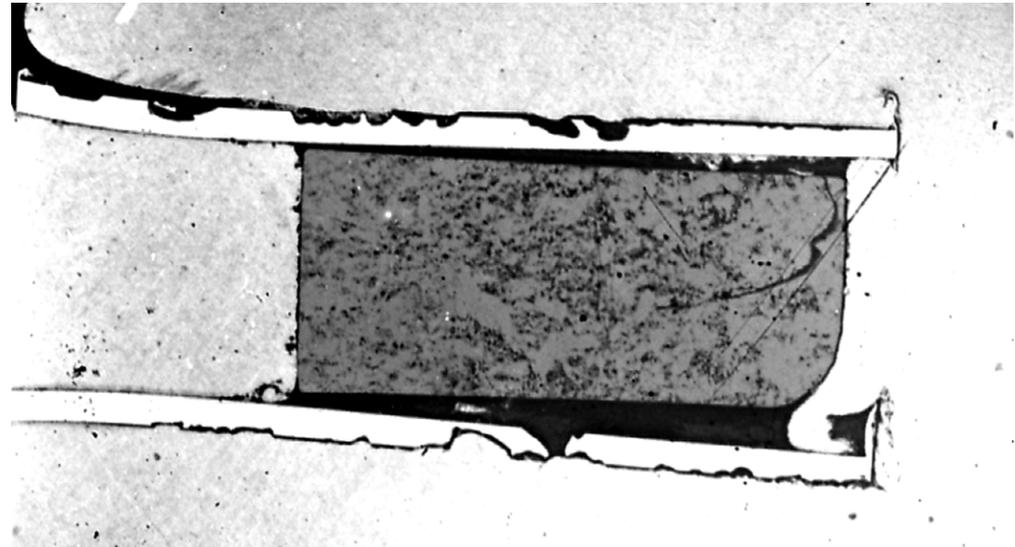
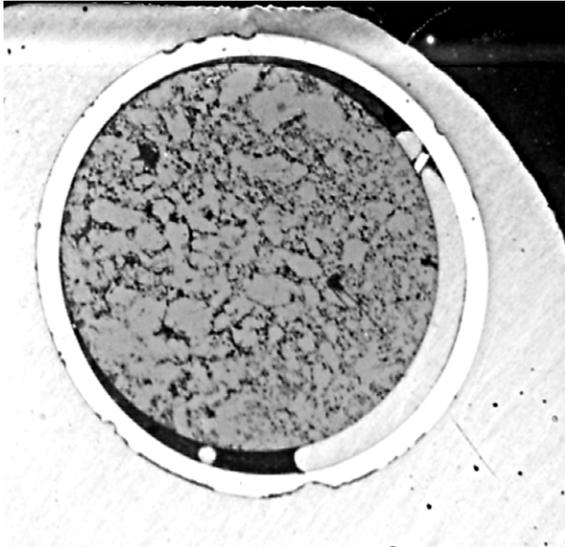
Temperature dependence of corrosion depth in EP-450 steel claddings at a burn-up of 10.8 % h.a.

Decarburization

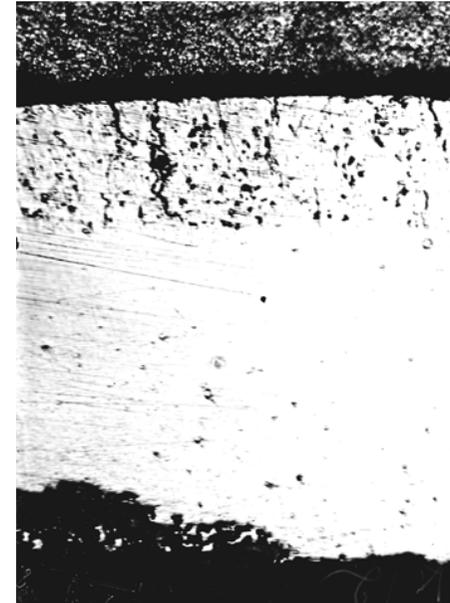
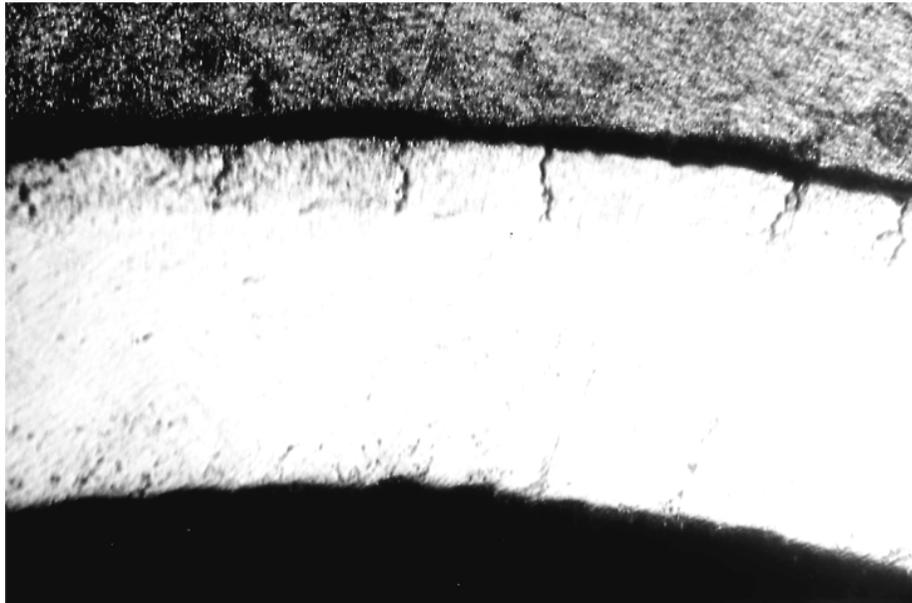


Decarburization of EP-450- steel claddings from the coolant side

Storage in cooling pool BOR-60



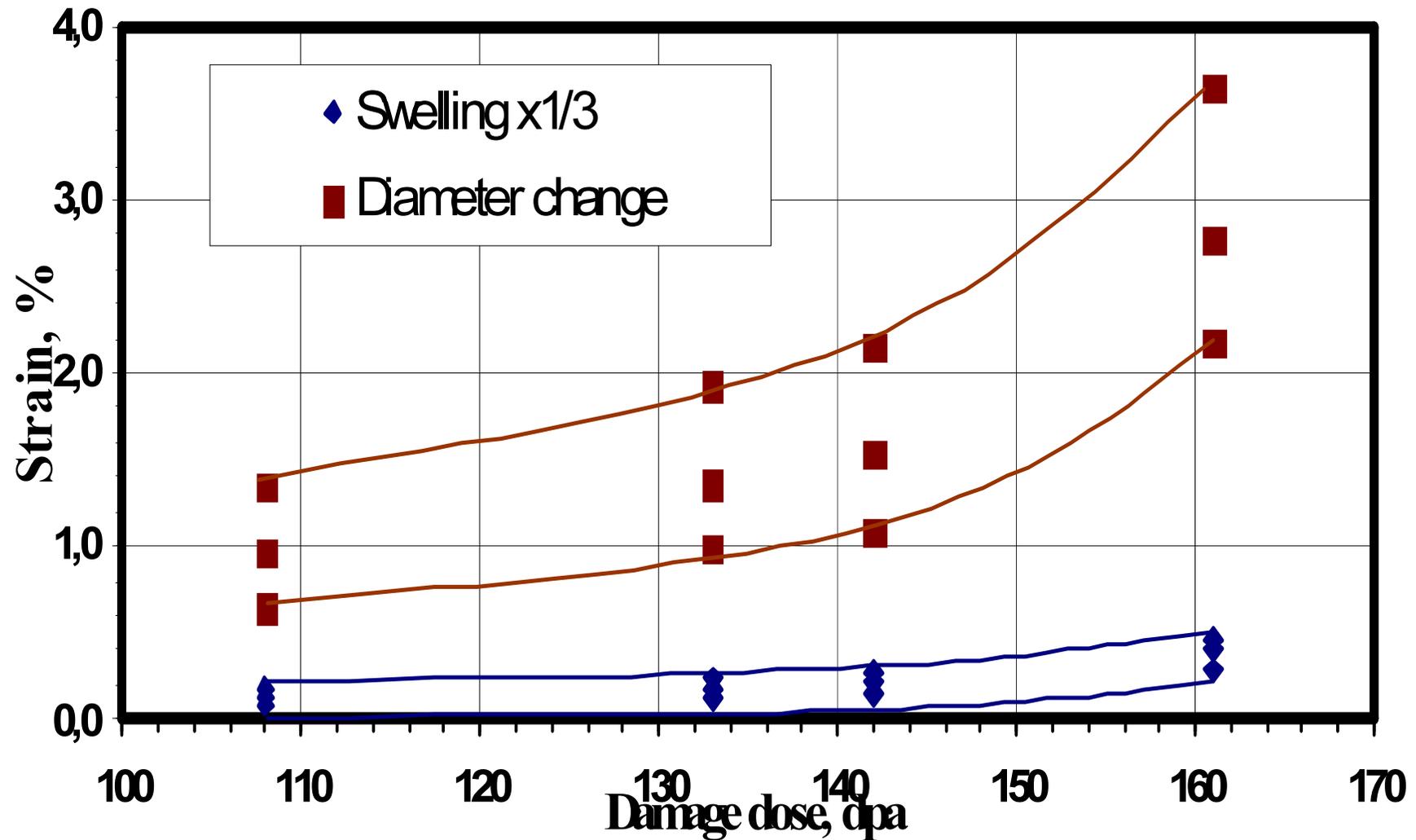
Storage in cooling pool BN-350



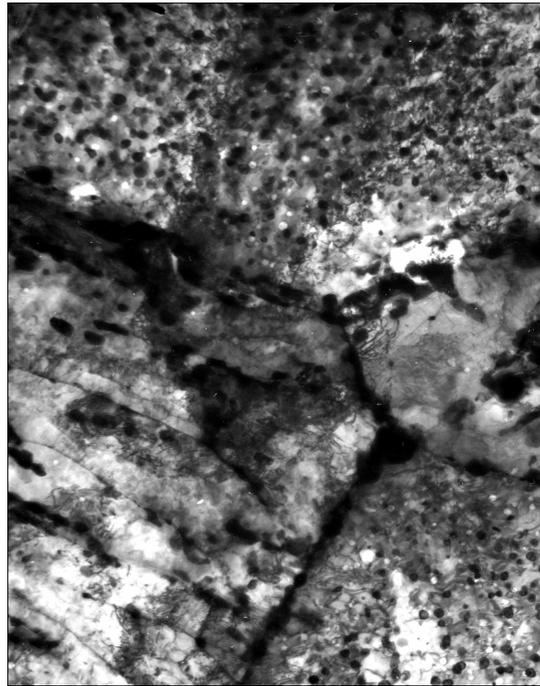
Irradiation parameters of fuel rods with EP-450 steel claddings and vibropacked fuel

FA index	Fuel	Burnup %, h.a.	Maximum damage dose, dpa	Maximum linear power, W/cm	Maximum and minimum temperature of cladding internal layer, °C
OG-115	(0.8U+0.2Pu)O ₂	21.0	~108	485	690 - 540
OG-116	(0.8U+0.2Pu)O ₂	24.4	~132	476	650 - 530
OG-117	(0.8U+0.2Pu)O ₂	27.8	~142	478	650 - 510
VS-007	UO ₂	30.0	~163	352	600 - 470

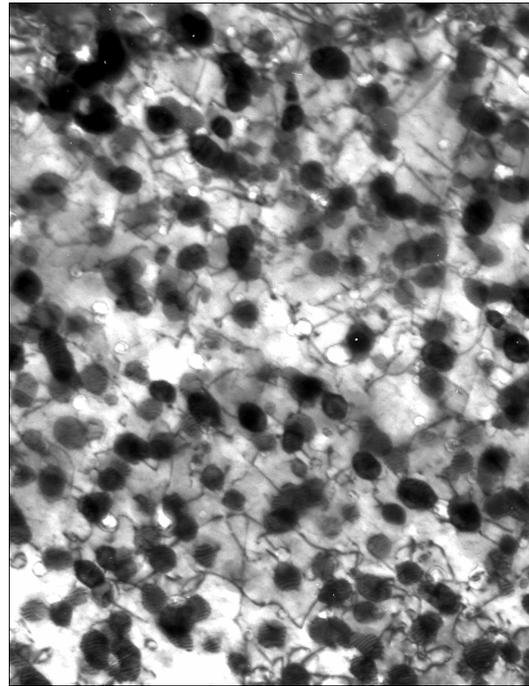
Maximum diametral strain of EP-450 steel claddings



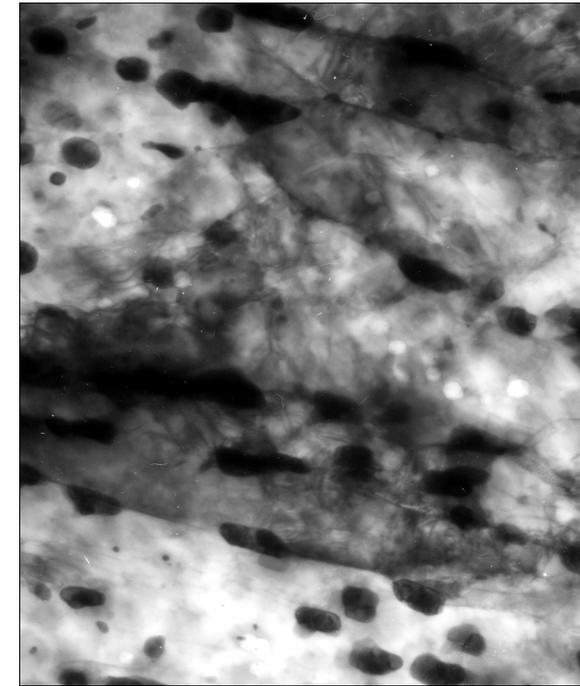
Structure of EP-450 steel



a



b



c

Structure of EP-450 steel as cladding material for BS-007 fuel rods irradiated in BOR-60 reactor (a – general view, x20000; b – voids, dislocation network and χ -phase precipitates in ferrite, x40000; c – voids and secondary phase precipitates in sorbite, x40000)



ИНЭП

Conclusions

- ✓ The EP-450 steel as a cladding material for fast reactor fuel rods retains its high resistance to radiation swelling at damage doses of up to 160 dpa.
 - ✓ The main factors hindering reliable performance of the fuel rods with EP-450 claddings are the maximum initial temperature and the level of stresses in the claddings caused by gas and fuel pressure.
 - ✓ The problems associated with heat strength and corrosion by the iodine transport reaction can be resolved by decreasing the maximum initial temperature down to 650°C.
 - ✓ Use of the EP-450 steel as a cladding material in the BN commercial reactors is impossible with the FA storage technology available now.
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