Nuclear Fuels and Nuclear Fuel Cycle

Nuclear Fuel Cycle and Materials Section Chaitanyamoy Ganguly Head

Seminar on Nuclear Science and

Technology for Diplomats

IAEA, Vienna, 3 February 2009



IAEA International Atomic Energy Agency































UNIQUE FEATURES OF NUCLEAR FUEL AND
NUCLEAR POWER REACTOR

High Energy Density :

1 atom of 'C' on combustion releases:	~ 4 eV
1 atom of U235 on fission release:	~200 MeV
1 kg of coal can generate:	3 kWh
1 kg of oil can generate:	4 kWh
1 kg of natural uranium (0.7% U-235):	50,000 kWh
1 kg of plutonium:	60,000,000 kWh
Annual Fuel Requirement of a 1000 MWe	Power Station:
Nuclear : 30 tons: Coal : 2.6 million tons	: Oil : 2.0 million tons

Environment Friendly: Zero emission of CO₂, SO₂ & NOx –does not contribute to global warming, climate change and acid rain







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CIVILIAN APPLICATION	MILITARY APPLICATION
URANIUM Natural Uranium (~0.7% U-235) Fuel for PHWR: UO2 pellets Uet for hays: UO2 pellets Uet for hays: water-cooled an moderated research reactor U-metal Low Enriched Uranium (LEU: <20% U-235) Uet for LWRs, AGRs, RBMK UO2 pellets containing <5% U-235 Fuel for LWRs, AGRs, RBMK UO2 pellets containing <5% U-235 Reprocessed Uranium (RepU: 0.8 - 1.1% U-235) from reprocessing spent fuel from thermal reactors Re-enrichment & use as LEU fuel in LWRs Use in CANDU reactor (DUPIC (Direct Utilization of PWR Spent Fuel In CANDU] Blanket material for LMFR Depleted Uranium (<0.7% U-235) [tailings of enrichment plant: 0.1% - 0.3% U-235 or reprocessed spent fuel from PHWRs using natural uranium oide fuel Use as blanket material in LMFR for breeding Pu-239 Use for neuton-fuel Attaining of initial core OPHWR Auting start up.	URANIUM • High Enriched Uranium (HEU) (>20% U-235) Weapon grade HEU: >90% U-23:
PLUTONIUM Civilian 'Pu' from spent thermal power reactors contain 55-70% Pu-239 (balance is Pu-240, Pu-241, Pu-424 Pu-238) could be used as 'Dirry Bomb' • Use as mixed uranium plutonium oxide (MOX) fuel for LWR, PHWR & LMFR • Use as advanced LMFR fuel as mixed uranium plutonium carbide, nitride and U-Pu-Zr alloy containing 15-25% Pu	PLUTONIUM • Weapon grade plutonium: >95% Pa-239 • Civilian plutonium could be used as a 'dirty' bomb









Is t	here enou (Ref: Re	igh Ura d Book 2007)	nium ?
Identified Resou Total Conventio Phosphates (und	nal Resourc	es	5.55 Mt 15.9 Mt 22 Mt (?)
Reactor/Fuel cycle	assuming u year (corresp	iranium cons onding to the	ium Resources will last umption of ~ 66,500 tons / e year 2006 when 2663 TWh y was generated)
	Using only Identified Resources	Using Total Conventiona Resources	
Current technology	100	300	> 675
Fast reactors with closed fuel cycle and recycling.	>2 500	>8 000	~20 000

















IAEA URANIUM PRODUCTION SITE APPRAISAL TEAM (UPSAT) -REVIVED IN 2008 UPSAT Guidelines -- IAEA-TECDOC-878(1996)





The objective of IAEA Uranium Production Site Appraisal Team (UPSAT) is to assist to improve the Operation & Safety of Uranium Production Sites , in Member States(on request) by peer review involving international experts.









Country	Facility	Capacity	Status
Brazil	Resende Enrichment	120	Commissioning
China	Lanzhou 2	500	In operation
China	Shaanxi Uranium Enrichment Plant	500	In operation
France	Eurodif (Georges Besse)	10 800	In operation
Germany	Urenco Germany GmbH	1 800	In operation
Japan	Rokkasho Uranium Enrichment Plant	1 050	In operation
Netherlands	Urenco Nederland	3 500	In operation
Pakistan	Kahuta	5	In operation
Russian Federation	Angarsk	1 000	In operation
Russian Federation	Siberian Chemical Combine (Seversk)	4 000	In operation
Russian Federation	Ekaterinburg (Sverdlovsk-44)	7 000	In operation
Russian Federation	Krasnoyarsk	3 000	In operation
United Kingdom	Urenco Capenhurst	4 000	In operation
United States of America	Paducah Gaseous Diffusion	11 300	In operation



	Operating Conversion Plants (Refin			
Country	Facility	Capacity	Status]
Argentina	Pilcaniyeu - 1	62	In operation	
Canada	Cameco - Port Hope (UF6)	12 500	In operation	
China	Lanzhou	400	In operation	1
France	Comurhex Pierrelatte (UF6)	14 000	In operation	
Russian Federation	Angarsk	20 000	In operation	
Russian Federation	Ekaterinburg (Sverdlovsk-44)	4 000	In operation	
United Kingdom	BNFL Springfields Line 4 Hex Plant	6 000	In operation	
United States of America	Metropolis / Converdyn	17 600	In operation	
	Total Capacity	74 562		
		74 302	tHM/year	
	Operating Conversion Plants (UF6/I eady for Pettetizing)			
UO2 Powder re	Operating Conversion Plants (UF6/I			
UO2 Powder re Country	Operating Conversion Plants (UF6/I ady for Pettetizing)	Refined Uran	ium Oxide to	
UO2 Powder re Country Argentina	Operating Conversion Plants (UF6/I sady for Pettetizing) Facility	Refined Uran	ium Oxide to Status	
UO2 Powder re Country Argentina Brazil	Operating Conversion Plants (UF6/l ady for Pettetizing) Facility Complejo Fabril Cordoba	Capacity 150	ium Oxide to Status In operation	
UO2 Powder re Country Argentina Brazil Canada	Operating Conversion Plants (UF6/l ady for Pettetizing) Facility Complejo Fabril Cordoba Fabrica de Combustivel Nuclear	Capacity 150 120	ium Oxide to Status In operation In operation	
UO2 Powder re Country Argentina Brazil Canada France	Operating Conversion Plants (UF6/I ady for Pettetizing) adj for Pettetizing) Complejo Fabril Cordoba Fabrica de Combustivel Nuclear Cameco - Port Hope (UO2)	Capacity 150 120 2 800	ium Oxide to Status In operation In operation In operation	
UO2 Powder re Country Argentina Brazil Canada France India	Operating Conversion Plants (UF6/I bady for Pettetizing) Facility Complej Fabril Cordoba Fabrica de Combustivel Nuclear Cameco - Pohop (UO2) TU2 Cogema	Capacity 150 2800 350	ium Oxide to Status In operation In operation In operation In operation In operation	
UO2 Powder re Country Argentina Brazil Canada France India Pakistan	Operating Conversion Plants (UF6/I add for Pettetizing) Facility Complejo Fabril Condoba Fabrica de Combustivel Nuclear Cameco - Port Hope (UO2) TUZ Cogeme NFC - Hyderabad (UOP)	Capacity 150 120 2 800 350 450	Status In operation	
	Operating Conversion Plants (UF6N aday for Pottettizing) Facility Complejo Fabril Cordoba Fabrica de Combustivel Nuclear Canneo - Port Hope (UO2) TUZ Cogerna NFC - Hyderabad (UOP) Liamabad	Capacity 150 2800 350 450 0	Status In operation	







Country	Facility	Capacity	Status
Argentina	Ezeiza - Nuclear Fuel Manufacture Plant	160	In operation
Canada	Peterborough Facility	1 200	In operation
Canada	Zircatec Precision Ind Port Hope	1 200	In operation
China	Candu Fuel Plant	200	In operation
India	NFC - Hyderabad (PHWR)	270	In operation
India	NFC - Hyderabad (PHWR)-2	300	In operation
Korea, Republic of	CANDU Fuel Fabrication Plant (2)	400	In operation
Pakistan	Chashma	20	In operation
Romania	Pitesti Fuel Fabrication Plant (FCN)	200	In operation
	4	3 950	tHM/year































Oxide P	uel for Light				wii 0/yea
	(Powd	er, Pellet and	Assemb	iy)	
Country	Operator	Facility	Powder	Pellet	Assembly
Belgium	AREVA NP EU	Dessel	0	700	700
Brazil	INB-Resende	FCN Resende	165	120	240
China	Jianzhong	Jianzhong	400	400	450
France	AREVA NP EU	Romans	1200	820	820
Germany	AREVA NP EU	Lingen Fab	650	650	650
India	NFC-Hyderabd	Hyderabad	48	48	48
Japan	NFI-Kum/Tok	Kumatori	0	360	284
	MNF-TokaiMur	Tokai MNF	475	440	440
	NFI-Kum/Tok	Tokai NFI	0	250	250
	JNF-Yokosuka	Yokosuka	0	620	750
Kazakhstan	Kazatomprom	Ulba	3000	1000	0
Russia	TVEL-Ele/Nov	Elemash	1000	850	785
	TVEL-Ele/Nov	Novosibirsk	150	150	1000
South Korea	KNFC-Daejeon	Daejeon	600	600	600
Spain	ENUSA-Juzbad	Juzbado	0	400	400
Sweden	WestSE-Vas	Vasteras	530	530	400
U.S.A.	WestUS-Colum	Columbia Fab	1350	1500	1500
	AREVA NP US	Lynchburg	0	0	700
	AREVA NP US	Richland	1800	700	700
	GNF-Wilmingt	Wilmington	1000	1100	1100
United Kinadom	WestUK-Sprin	Springfields	440	440	0
Total		MTU/Year	12808	11678	11817







Country	Facility	Capacity	Status
France	La Hague - UP2-800	1000	In operation
France	La Hague - UP3	1000	In operation
India	Plutonium Plant, Trombay		In operation
India	Prefre-2, Kalpakkam		In operation
India	Prefre-1, Tarapur		In operation
Russian Federation	RT-1, Combined Mayak	400	In operation
United Kingdom	BNFL B205 Magnox Reprocessing	1500	In operation
United Kingdom	BNFL Thorp	900	In operation
	Total Capacity	4 800	tHM/year















Advanced Fuels for Liquid Metal-cooled Fast Reactors (LMFR)								
Properties	(U _{0.8} Pu _{0.2})O ₂	(U _{0.8} Pu _{0.2})C	(U _{0.8} Pu _{0.2})N	U-19Pu-10Zr	-			
Theoretical Density g/cc	11.04	13.58	14.32	15.73				
Melting point ^o K	3083	2750	3070	1400				
Thermal conductivity (W/m ºK) 1000 K 2000 K	2.6 2.4	18.8 21.2	15.8 20.1	40				
Crystal structure	Fluorite	NaCl	NaCl	Ŷ				
Breeding ratio	1.1 - 1.15	1.2 - 1.25	1.2 - 1.25	1.35 - 1.4				
Swelling	Moderate	High	High (?)	High				
Handling	Easy	pyrophoric	Inert atmos Inert atmos					
Compatibility - clad coolant	average average	Carburisation good	good good	eutectics good				
Dissolution & reprocessing amenability	Good	Demonstrated	risk of C ¹⁴	Pyro- reprocessing				
Fabrication/Irradiation experience	Large Good	limited	very little	limited				





Summary

- Natural uranium and natural thorium are the basic raw materials for nuclear fuels .
- Light water reactors (LWR) account for more than 80% of the operating nuclear power reactors, followed by pressurized heavy water reactor (PHWR) which contribute to ~ 6% of power reactors. LWRs will dominate the nuclear power worldwide up to 2050 and beyond.
- The present generation of nuclear power reactors use Uranium-235 as fuel.
 U-235 is the only fissile material in nature. The LWRs use LEU containing < 5% U-235 and the PHWRs use natural uranium as fuel. The plutonium 239 formed in the reactor by neutron capture reaction of fertile 238 is a man- made fissile material. In operating water cooled reactors, in-situ fission of Pu239 contribute ~ 30% of fission heat energy.
- Uranium resources are more or less uniformly distributed in the world and are adequate to meet any foreseeable growth scenario of nuclear power
- Uranium Mining & Milling, Conversion, Enrichment, and Fabrication of Uranium Oxide Powder, Pellets and Zirconium alloy clad Uranium Oxide Fuel Assemblies are being carried out on an industrial scale in several countries

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Summary - cont'd

- The operating nuclear power reactors utilize only 1% of the mined uranium the rest is stored in the form of depleted and reprocessed uranium mostly as U-238. The U-238 could be utilized for breeding plutonium in a fast reactor. Plutonium is the best fissile material for fast reactor.
- Natural thorium has no fissile isotopes. Neutron capture reaction with thorium lead to the formation of man –made fissile isotope U-233. Thorium based fuel must contain a fissile isotope (U-235, Pu-239 or U-233). Though thorium is three times more abundant in nature compared to uranium, thorium - based fuels are not likely to be commercialized till the uranium resources are utilized. Thorium 232- uranium 233 fuel cycle is best for thermal neutron reactors.
- The fissile isotopes (e.g. U-235, Pu-239, U-233 etc) are dual use materials having both civil and military applications. Proliferation -resistance in nuclear fuel cycle is essential to avoid clandestine diversion of fissile materials for non peaceful purpose
- Nuclear Fuel Cycle activities are matured industries in several countries in the world

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Through:

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- technical co-operation organizing technical meetings, symposia and coordinated research projects preparation of state -of -the art technical documents maintaining & updating databases on nuclear fuels and fuel cycles
- 3. 4.

