



# Improvements in defense in depth on French NPPs following Fukushima Accidents

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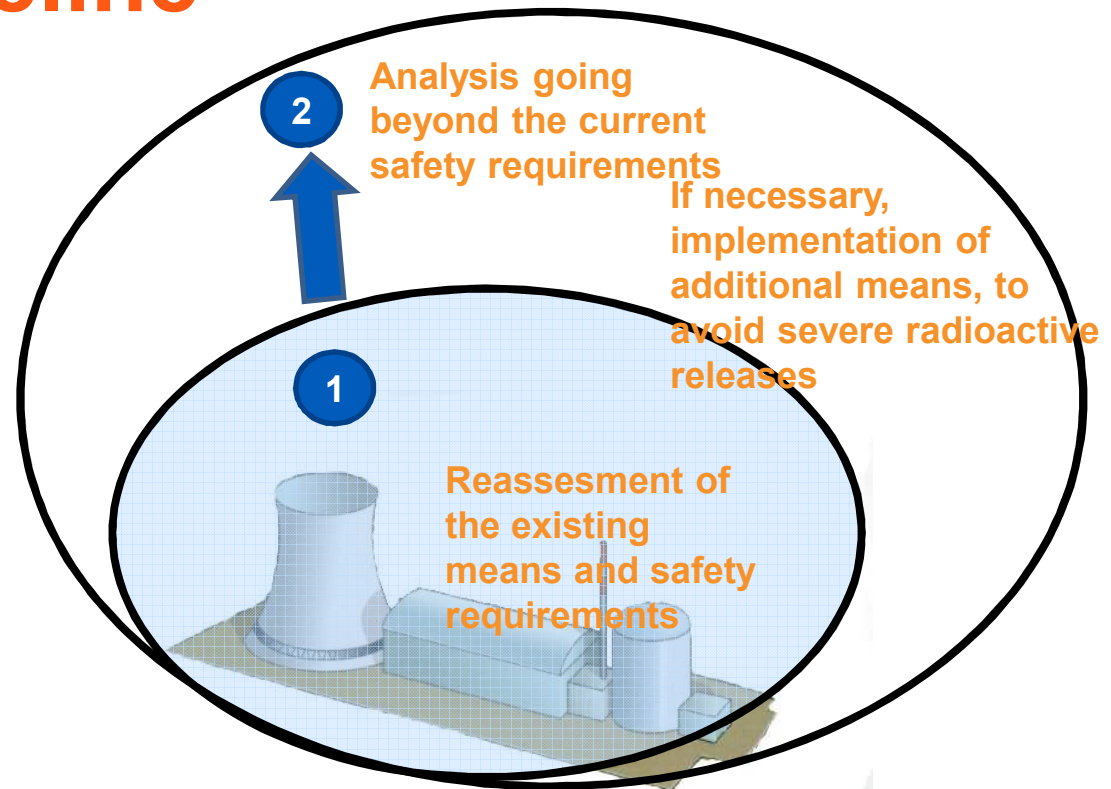
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# Context - Timeline

## 6 fields assessed :

- Earthquake
- Flooding
- Loss of heat sink LHS
- Loss of electrical supply SBO
- Severe accident management
- Sub-contracting



## Short time process

Required by ASN May 5th, 2011

Completed by EDF September 14th

ASN report January 3rd, 2012

European peer review conclusions April 25th

ASN Prescriptions, June 26 th

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# Earthquake

## Regulatory approach

- Deterministic (RFS 2001-01)
- Historical Research
- Translation near the site → SMHV
- Margin of 1 MSK → SMS
- Margin due to series effect on standard equipment
- Upgrade on plants during PSR (due to update of data & changes in requirements)

### **Existing margin evaluations**

- SMA for Tricastin
- PSA for St Alban

## CSA

- Evaluation to 1.5 SMS
- Around 200 components (those needed in SBO)
- Walkdown on each reactors
- By teams trained to SMA methods

## Main Conclusions

- Compliance ensured though PSR
- Margins up to at least 1.5 SMS on most components
- Additional evaluations required
  
- ENSREG recommends to implement probabilistic methods

# Flooding

## 1/2

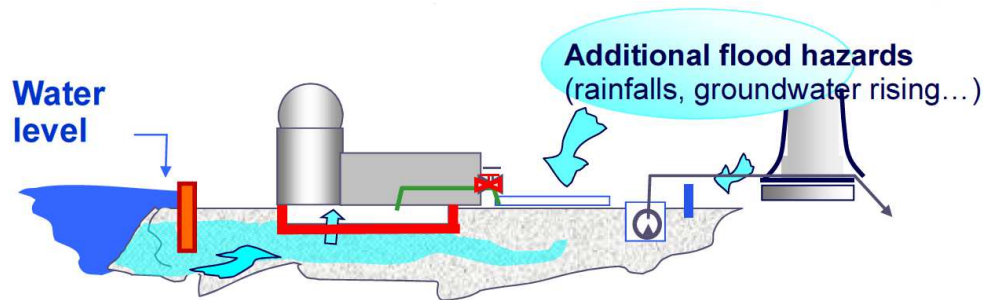
### Initial Regulatory approach : RFS 1.2.e

- CMS level according to site type
- River side
  - 115 % of millennial flow rate + uncert.
  - Or upstream dam failure + centennial flow rate
- Sea side
  - Max tide + millennial storm surge + uncertainties

### Blayais Flooding Feedback (1999) : Full Review + additional phenomena

- Wind-waves on sea, river or channel
- Swelling due to operation of valves or pumps
- Rainfalls : intense and short, or long duration
- Water retaining structures (other than dams) deterioration
- Circuits or equipment failure
- Groundwater rise

# Flooding 2/2



## CSA

- Fixed coefficient load increase :
  - 130 % of CMS on river side
  - 1 m more on sea side
  - Rainfall x 2
  - Additional failures on platform

## Main Conclusions

- Compliance ensured after already decided works
- Additional protection to avoid LUHS, LOOP or SBO on sites which do not remain dry with load increase
- maintenance of the “volumetric protection” to be improved
- ENSREG recommends a comparative evaluation between DBF defined according to ASN requirements and those used in other countries

# Loss of Functions : LUHS, SBO

## Current Design

- LOOP : 3.5 days of fuel
- LUHS
  - Water reserves to supply SG
  - 100 h for 1 unit
  - 24 h for a whole costal site
  - 60 h for a whole riverside site
- SBO
  - EFW turbine driven pump
  - Emergency turbine generator
  - Minimum I& C
  - Injection to RCP seals
  - Autonomy > 24 h

## CSA

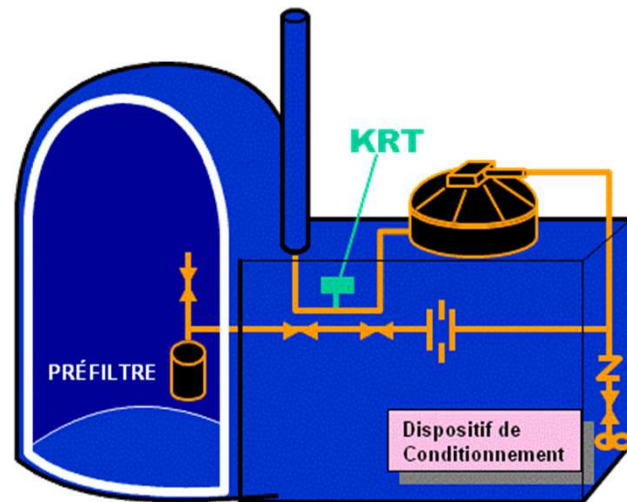
- No recovery of normal means
- All plant states, even rare
- Combination with external hazards

## Main Conclusions

- Water reserves not earthquake resistant
  - Additional means
- Sensitivity to turbine reliability
  - Additional generator sets
- Sensitivity of seals to high temps to be checked by tests

# Severe accidents

## Containment Filtered Venting



Pre-filter DF aerosols >10  
Filter DF aerosols >100  
DF I<sub>2</sub> > 10

- Pre-filter is earthquake resistant
- Filter shared between twin 900 MW units

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## Other Devices

- Passive Autocatalytic Recombiners
  - Limit H<sub>2</sub> concentration to 8 %
  - 2 with thermocouples
- Reliable RCP depressurisation
- Detection of RPV failure with thermocouples in reactor pit

## To be Installed

- Additional electrical supply of double-wall containment venting and control room venting systems
- Sodium tetraborate baskets in reactor building sumps (4 loop plants)

# Modifications of the design and of the organisation

Earthquake

Flood and other hazard

LHS/ SBO Reactor

LHS /SBO SFP

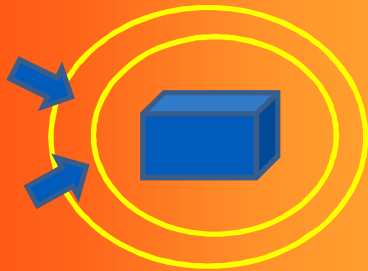
Severe accidents

*Protect the key safety functions*

*Prevention of Core Melt*

*Mitigation of major releases*

1. Reinforcement of protections against hazards



2. Additional electricity source

Additional water reserves

Severe Accident primary injection



3. Protective measures in case of core meltdown

Studies / knowledge of phenomena



Reinforcement of the crisis organization & management  
Ressources, fixed & mobile Equipment  
Emergency Crisis Centre + FARN





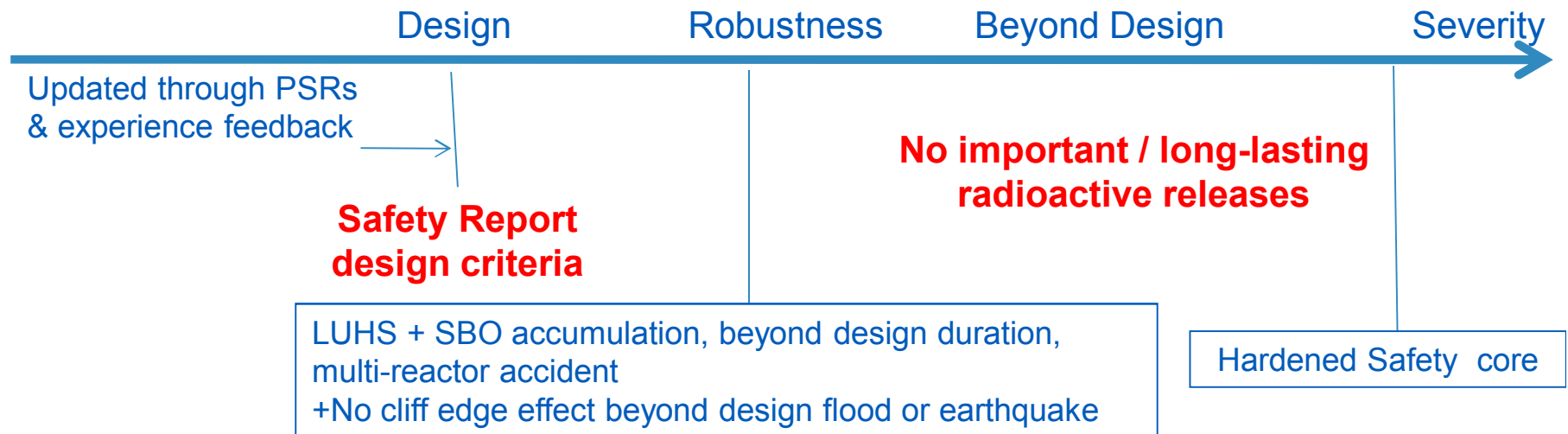
# HARDENED SAFETY CORE

## French Nuclear Safety Authority (ASN) Requirements :

“Before 30<sup>th</sup> June 2012, the operator will propose to the ASN a hardened safety core (HSC) of equipments and strengthened organisation, for extreme situations considered in post-Fukushima stress tests :

- to prevent a severe accident or limit its development,
- to limit a severe radioactive release
- to enable the operator to manage the emergency situations.”

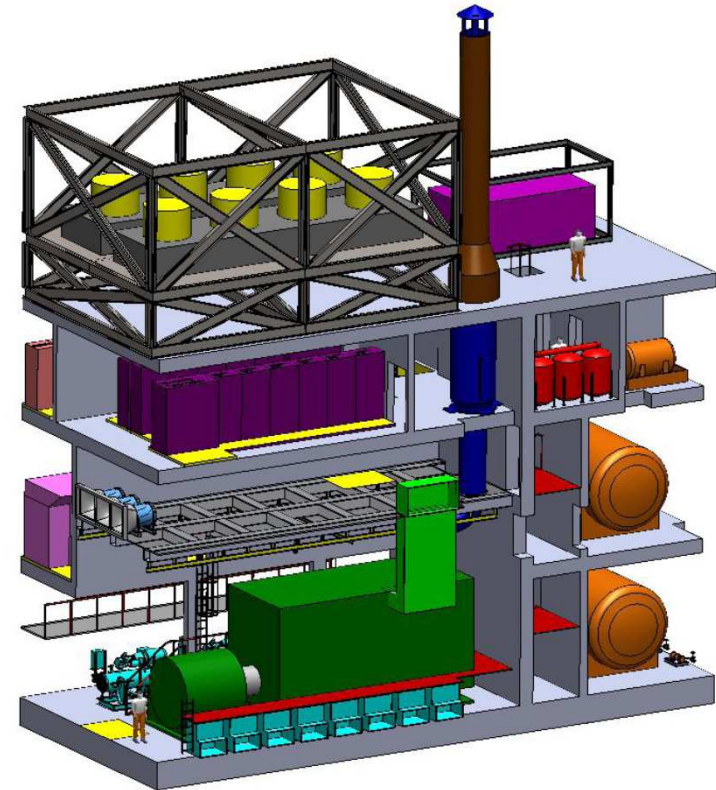
## Position in safety demonstration



# HARDENED SAFETY CORE

## Limited Key Functions

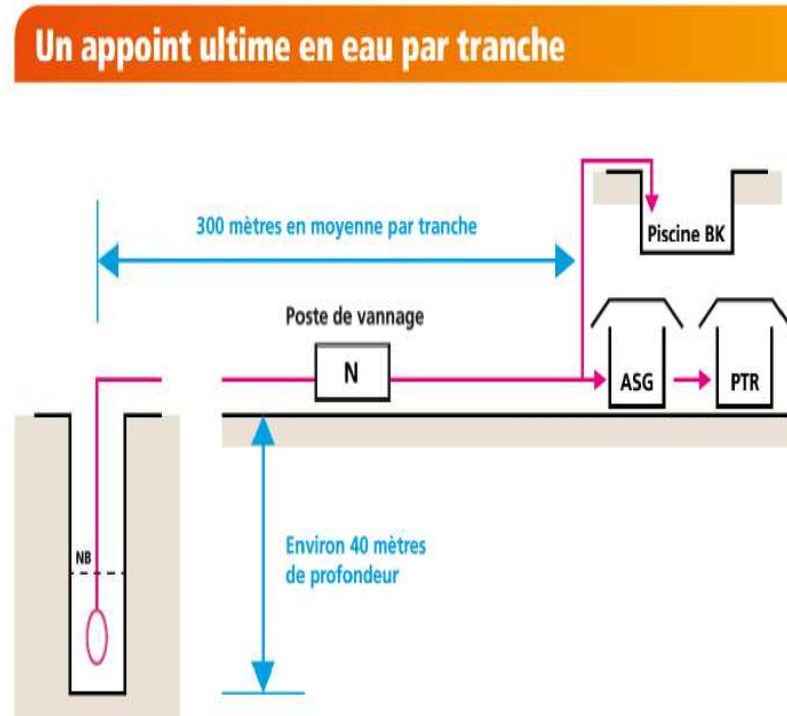
- Ultimate Additional Back-up Diesel, batteries and electrical connections
- Instrumentation (state diagnosis, radiological releases Control of HSC)
- Diversified emergency feed water
- Depressurization of RCS, and sufficient injection capacity
- Water make-up to spent fuel and reactor pools, and primary circuit
- Containment isolation
- Existing CFV
- New crisis management premises
- Mobile devices and essential means of communication



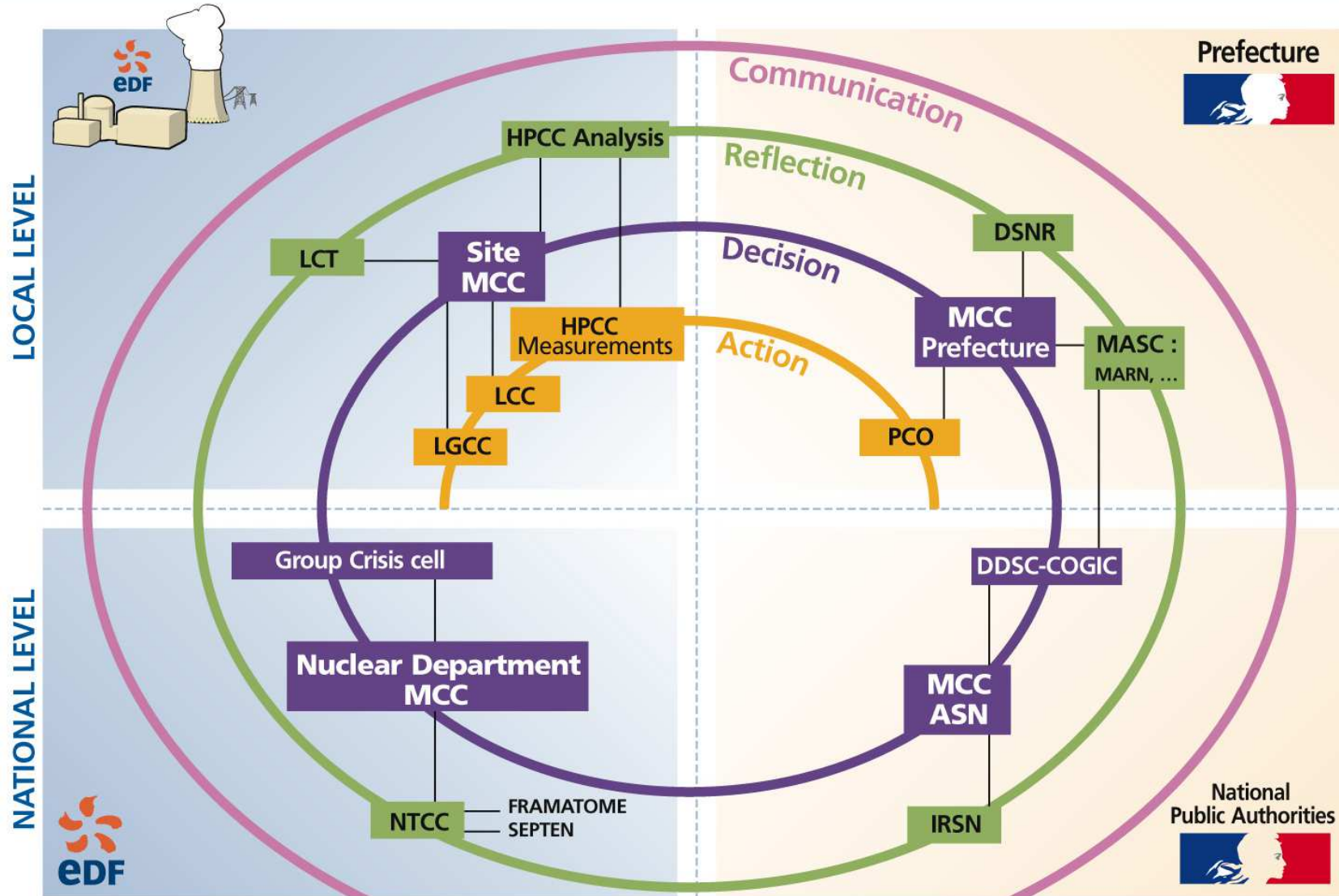
# HARDENED SAFETY CORE

## Extreme Load Cases

- Earthquake greater than 1.5 SMS, up to 0.4g (already being discussed)
- Flooding : Values used for CSA
  - + Additional margins for 2 sites
- F4 Tornado
- Lightning : 300 kA
- Hail : 50 mm, 32 m/s
- Induced effects of these loads
  - On other parts of NPP
  - On dangerous installation in vicinity



# CURRENT CRISIS ORGANISATION



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# CRISIS ORGANISATION

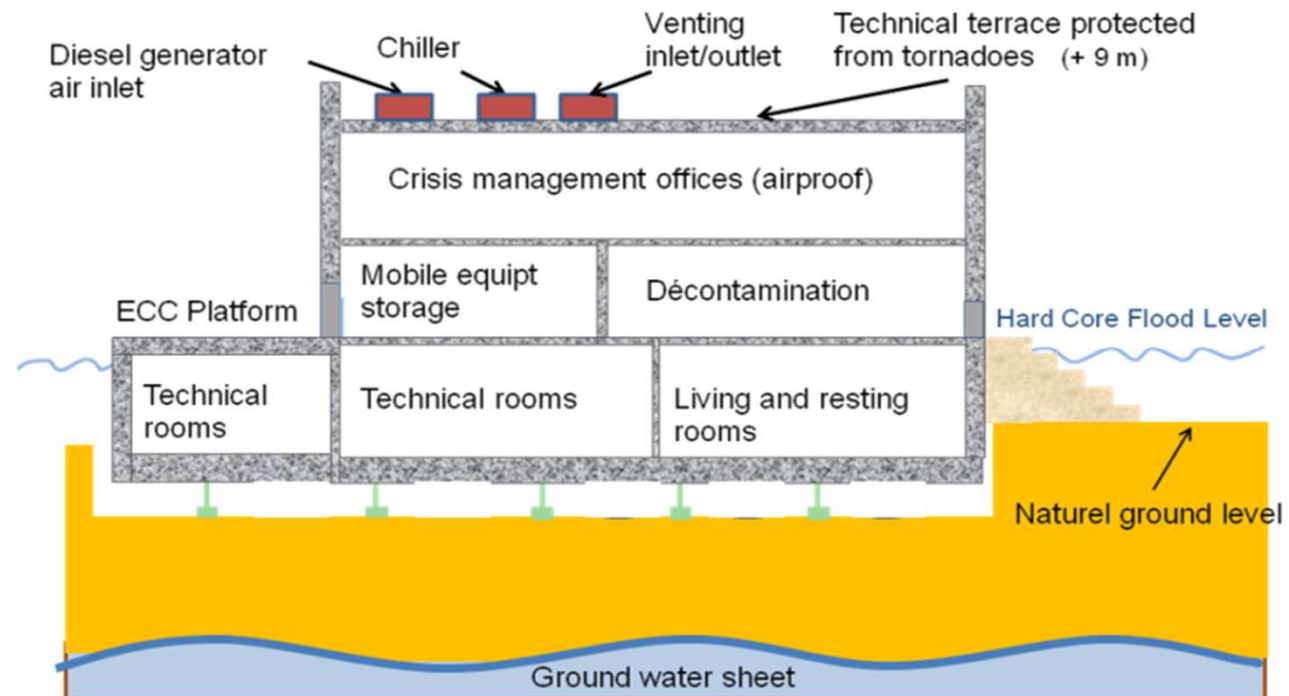
## Current Crisis organization

- has trained teams available on-call :
  - 350 people on each site
  - 300 people at national level (including Engineering)
- is periodically tested :
  - 250 drills per year for the whole EDF nuclear fleet
  - Up to 12 drills/site/year
  - 12 national scale drills (4 to 5 with public authorities)



# CRISIS ORGANISATION REINFORCEMENT

- In several steps
- To cope with multi-unit events
- To integrate FARN
- To reinforce local level capacity & protection



New Emergency  
Crisis Center  
(100 people  
capacity)

# FARN (Nuclear Rapid Response Force)

- Announced by EDF in April 2011
- Now Required by ASN (June 2012)

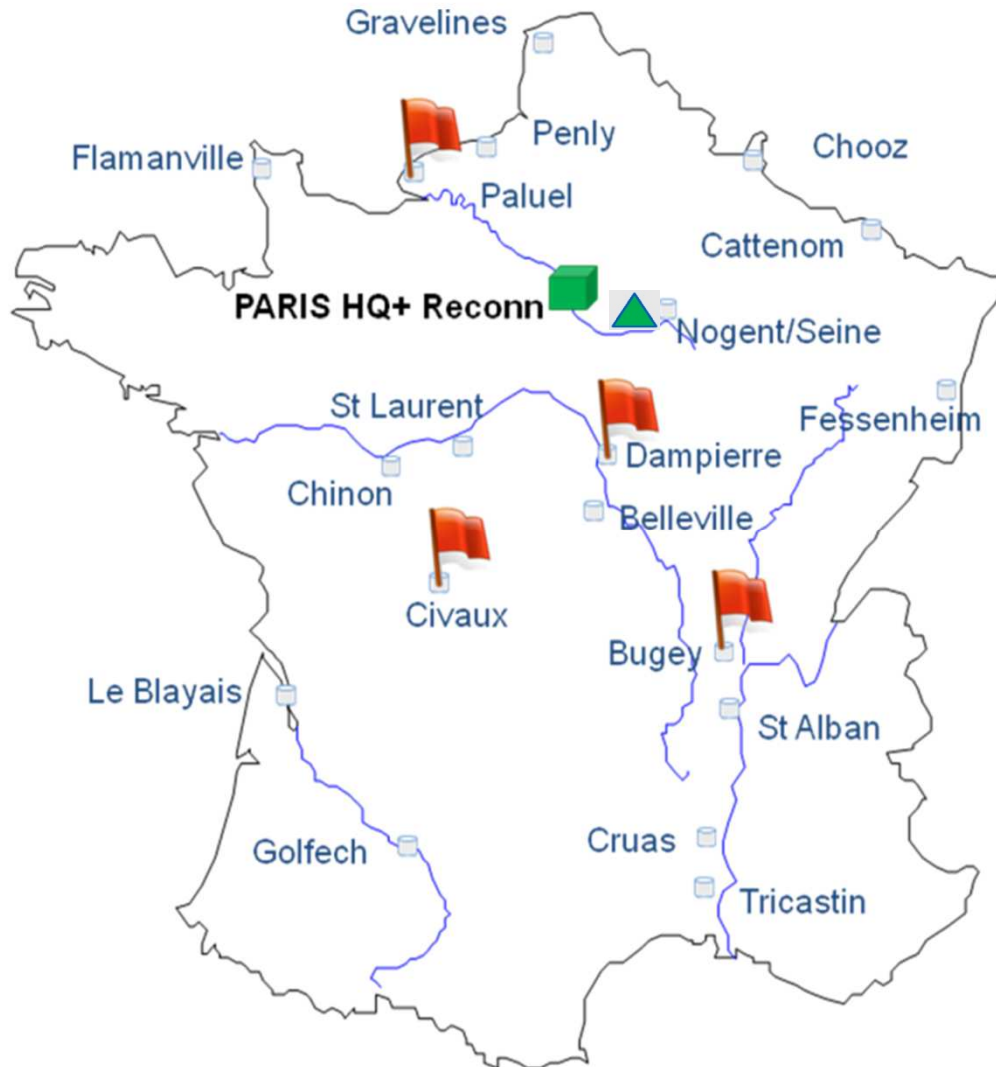
## Main hypotheses




- One site experience severe accident
- Major destructions in infrastructures
  - Including access roads
- Local teams may have unavailability
- Possible radiological or chemical hazard

## 3 steps of intervention

- In less than 12 h
  - Bring skilled operators
  - Evaluation of situation
  - Assistance to NPP using local means (fixed or mobile), up to 24h
- After 24 h
  - Brings its own mobile equipment and other resources
  - To guarantee site autonomy > 72h
- After 72 h
  - Additional resources from EDF group, or shared between French Nuclear Operators
  - To guarantee durable safe situation

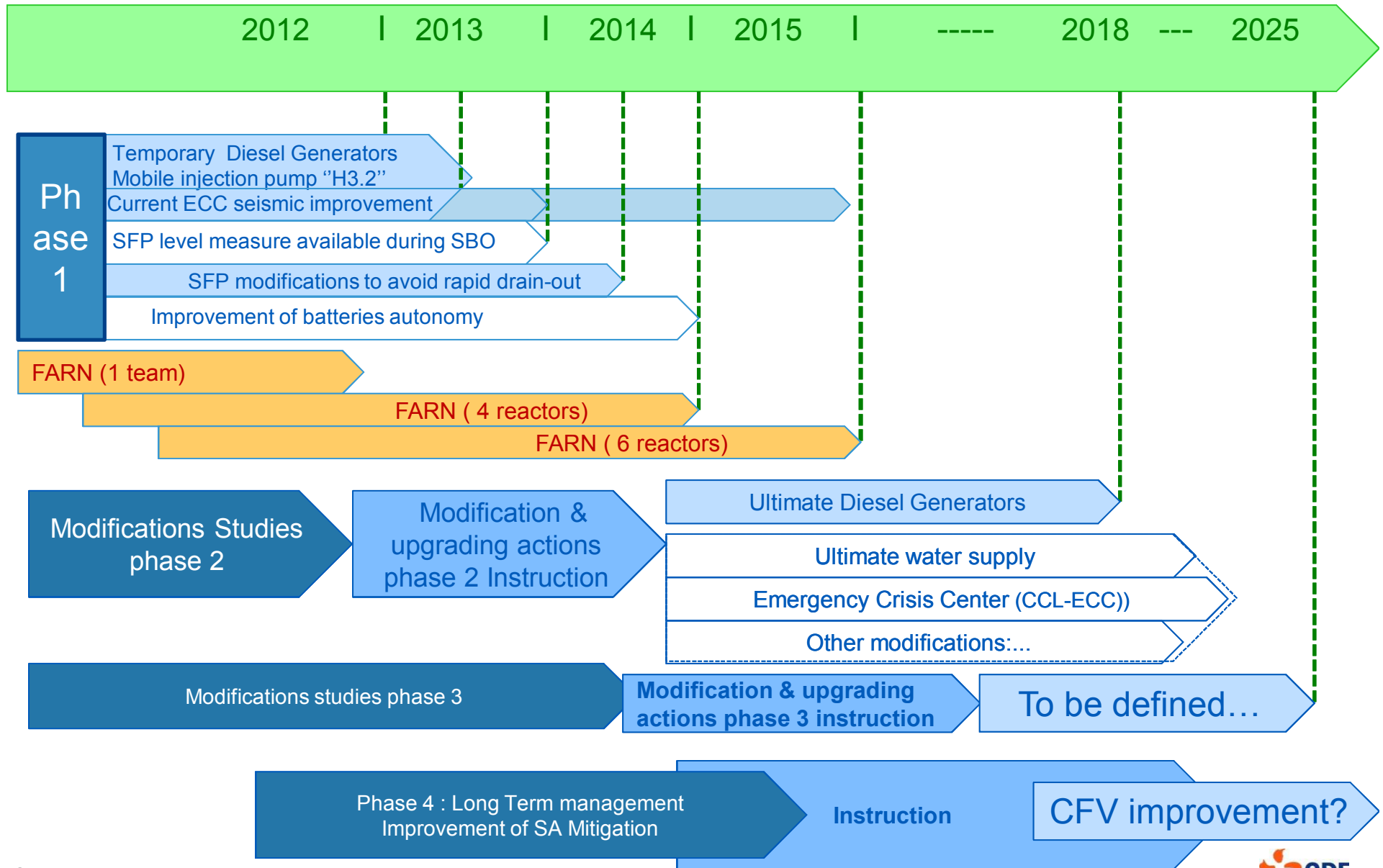
# FARN ORGANISATION



- 1 national headquarters 
  - (reconnaissance team, about 30 people in 5 on-call teams, country wide intervention)
- 1 national equipment base 
  - (long term equipment, rear bases modules)
- 4 regional FARN bases with  regional equipment bases nearby
  - hosted by 4 Farn'ed NPPs
  - each with about 70 people in 5 teams of 14 on-call people
- About 4 local rear bases predefined for each of the 19's NPPs
  - one to be chosen in case of severe situation on the NPP



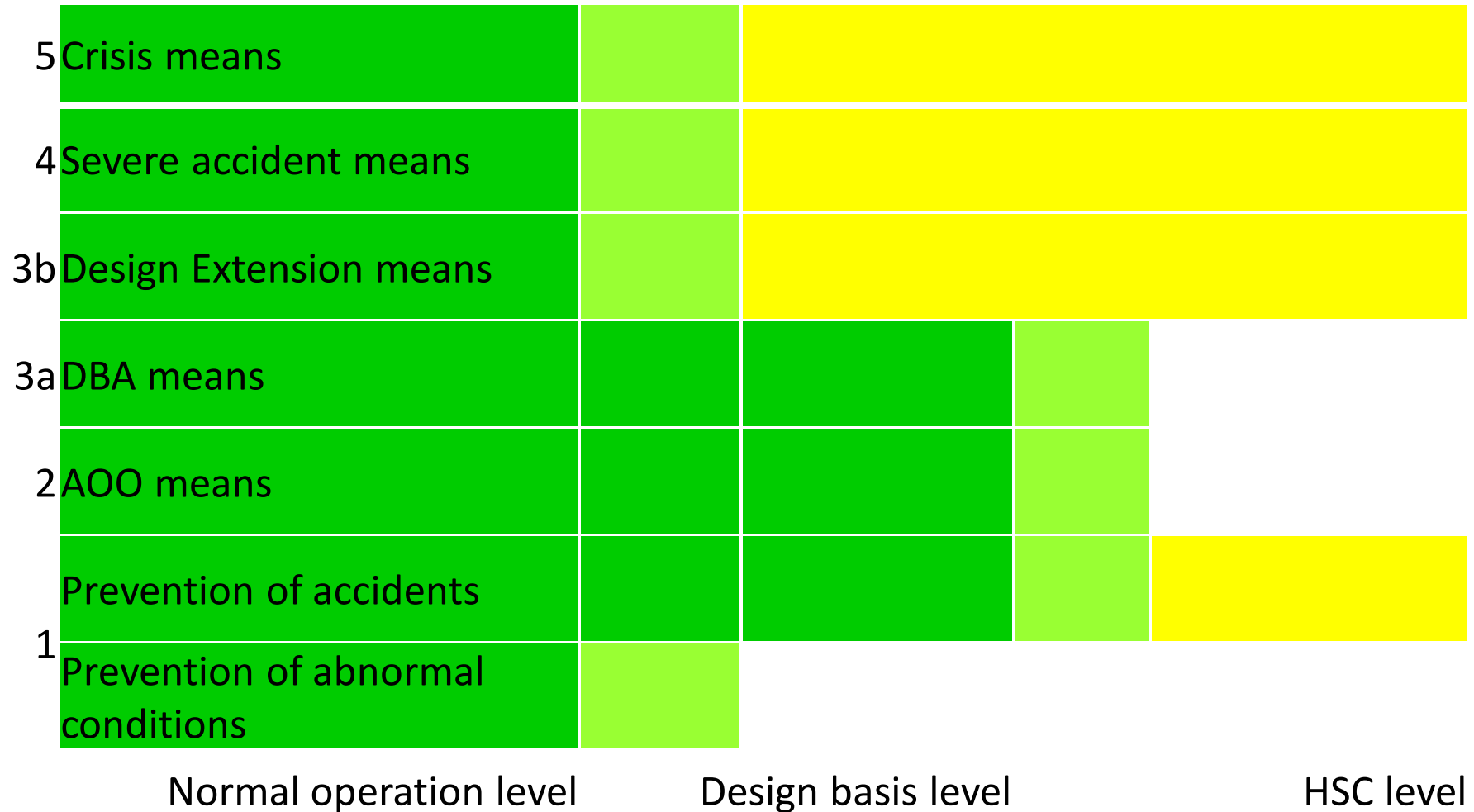
# General schedule – Most typical improvements deadlines



## Defence in Depth Concept

- ▶ **IAEA SSR 2-1 and Wenra : 5 levels**
- ▶ **Level 1 : Prevents deviations and failures :**
  - Prevention of AOOs
  - Prevention of accidents which result from failures
- ▶ **Level 2 : Prevents escalation from AOOs to accidents**
- ▶ **Level 3 : Prevention of core damage and releases from accidents. Subdivision from WENRA**
  - 3a : Generally single event such as pipe failure
  - 3b : Multiple failure events due to CCF
- ▶ **Level 4 : Prevention of large releases from core melt accident**
- ▶ **Level 5 : Mitigate radiological consequences of releases**

# Improvements in Did resulting from modifications



Initial requirement



Margins



Improvements

## The fundamentals of the robustness of EDF fleet are reinforced by the CSA

- ▶ **Initial design (PWR)** - good intrinsic robustness
- ▶ **Standardization of the EDF fleet:** 1500 reactor year operating feedback, homogeneity of the safety level
- ▶ **Continuous improvement of safety:** periodic safety reviews; experience taken from national events (Blayais in 1999, heat wave in 2003...) and international events (TMI, Tchernobyl)
- ▶ **Quality of plant operation** : operation, maintenance, emergency preparedness; transparency (deviation reporting)
- ▶ **Industrial organization and resources:** “architect engineer” model, integrated engineering
- ▶ **General industrial context** , process of international supplier selection (qualification and monitoring)

# Conclusion

- ▶ EDF, acting as designer and operator of its fleet, issued the CSA in a very tight schedule
- ▶ Following those CSA, EDF confirms **the present good level of safety** for all its nuclear reactors
- ▶ Following the **CSA new analyses**, EDF proposes **supplementary measures**, taking into account **potential extreme situations on a deterministic basis**
  - Modifications, new equipments: “hardened safety core” (a limited number of key safety SSC)
  - Organization : Nuclear Rapid Response Force, reinforcement of crisis management organization
- ▶ These analyses and modifications will continue **to improve even more the defense in depth of EDF’s nuclear fleet**