

Advanced Real-Time Control Systems

For

Magnetically Confined Fusion Plasmas

Presented by

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On behalf of

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Association EURATOM-IST

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Preamble

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Real Time Measurement and Control (RTMC) of magnetically confined plasmas is a critical issue for safe operation and high performance scientific exploitation of experimental devices on regimes beyond the current operation frontiers.

It is a broad task undertaken by most relevant devices and involving a large number of competencies:

-plasma physics, control, machine operations, systems, machine safety, software, hardware, communications engineering, computational mathematics, ...

An interdisciplinary dialogue is required at several levels to integrate, consolidate and build-on all the expertise and knowledge available for the benefit of ITER.

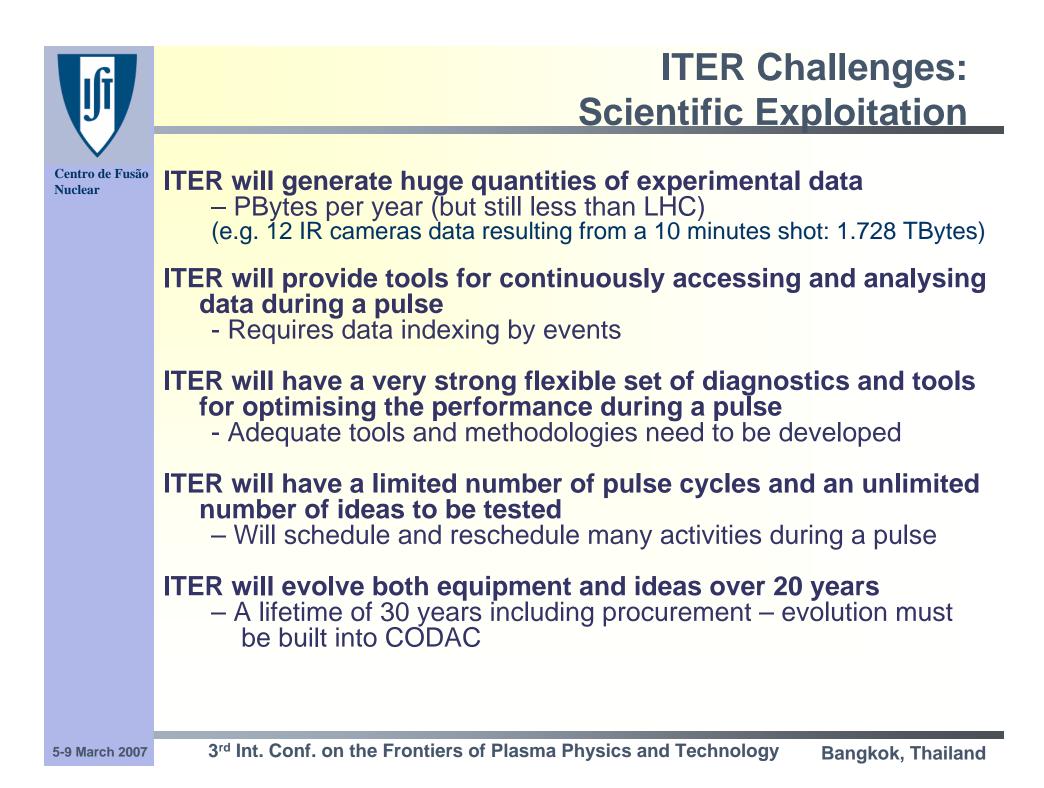
Plasma control is essential for achieving the goals of the ITER experimental programme

ITER Challenges: key areas Plasma Control involves very many aspects and ALL

must be addressed in a consistent and timely manner

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> to allow an adequate model-based control design **Physics** \Rightarrow The Objective Experiment **Modelling** \Rightarrow The Description Theory & modelling of physics and engineering **Control Engineering** \Rightarrow The Solution System specification, sensors, actuators, algorithms Control system software and hardware design Practical Implementation & Validation **Instrumentation and Information Technology** \Rightarrow The Environment **CODAC** standards Signal IO: ADCs, DACs, Processors and operating systems, etc. Communications: process (continuous) & supervisory (occasional) Databases

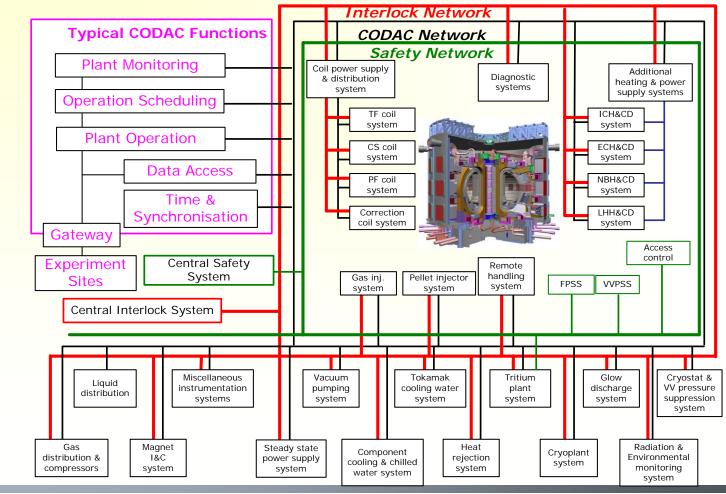


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ITER: as seen by CODAC

- CODAC integrates ALL ITER Plant Systems
- Many networks: operation, interlocks, safety
- CODAC functions are like present tokamaks



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ITER Challenges: RT and CODAC

Centro de Fusão Nuclear RT control systems have specific technical issues which are different from the more general (and much wider) CODAC issues, though similar solutions are likely to be pursued.

The ITER CODAC conceptual design is advancing towards peer review (not yet proposing technical solutions which would most likely be obsolete some years from now).

However, prototyping ideas is desirable and ALL concepts must be fully functional and exhaustively tested, well before it is essential for ITER

ALL CODAC functions/RT control systems must be proven on advanced facilities – existing tokamaks and/or ITER test stands

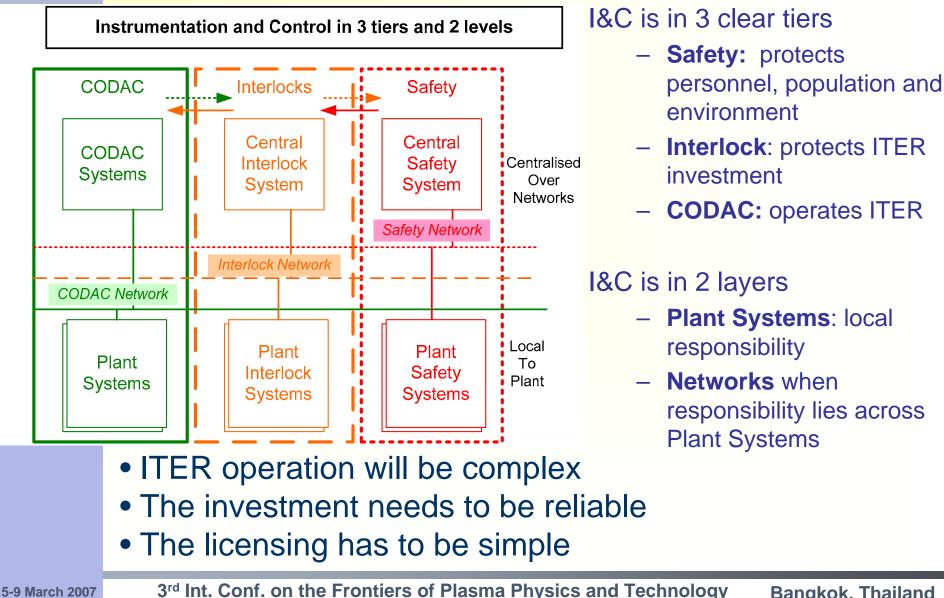
Near-term needs for RT control on existing devices may address issues of relevance for ITER-CODAC

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Instrumentation and Control



Bangkok, Thailand



ITER Challenges: Instrumentation & IT

Issues which need to be addressed for ITER

Assess design constraints and define requirements Define standards and guidelines to avoid proliferation of solutions and technologies

- Which technologies, protocols, methodologies
 - technical integration and inter-operability ?
 - multi-party, international development?

Design, develop and test appropriate solutions

Assess costs

Identify fusion specific needs

Choose long-life technologies for high availability and reduced maintenance

Dedicated time and funding will be required to address these issues

ITER Challenges: Inherent Constraints

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ITER site will be a Nuclear Registered site

- ITER environment between JET and a Nuclear power station in terms of quality, audit trails, etc.)
- To accomplish the scientific objectives will need control system architecture with freedom but within the licensing constraints
- Requires tools to guarantee safety, protection of investment (and a unique facility) and guaranteed operation
- Hostile environment for measurements, networks, electronics human access will be restricted
- ITER will require a far higher level of availability and reliability than previous/existing tokamaks

IT security will be a major issue

• Security with transparency is required. What does this mean for Remote participation?



ITER is assumed to be totally legacy-free for hardware and software

 Methodologies will have to be tested and proved on existing fusion devices before implementation on ITER

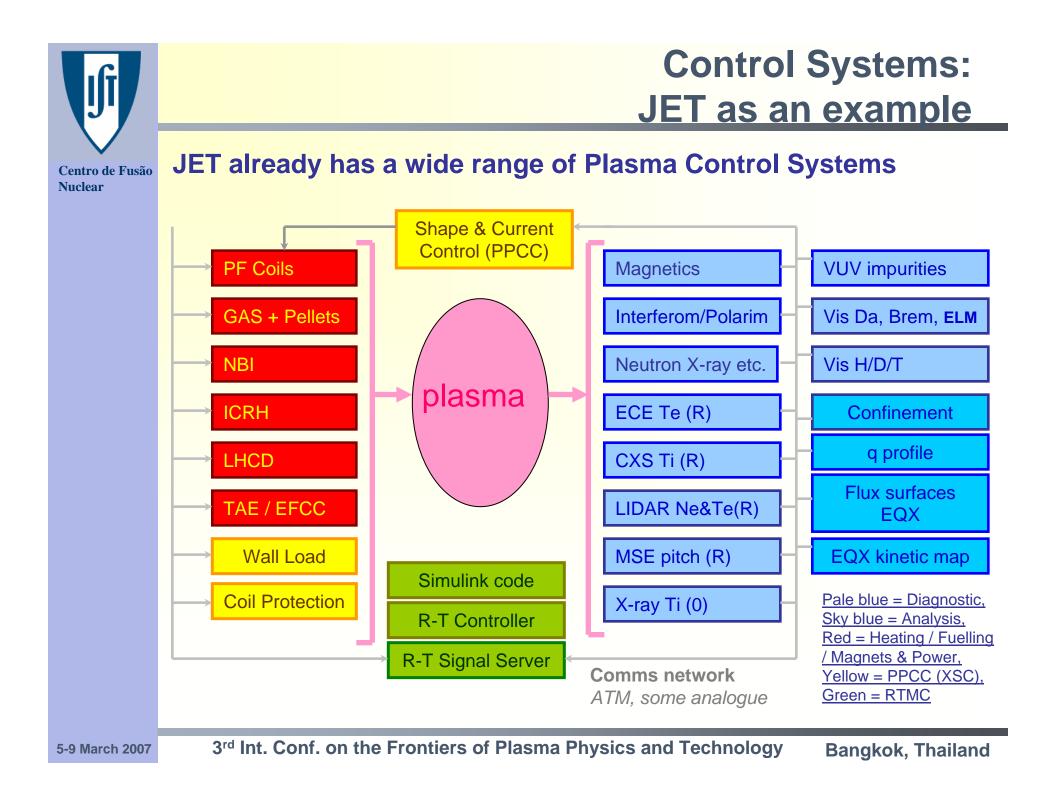
ITER Challenges:

Inherent Constraints

- It will be necessary to take informed decisions based on technology progress
- Maintenance will be an issue and proliferation of technologies must be avoided

ITER is an international project

- In-kind procurement world-wide
- Integration of Plant Systems from all participants
- The implications of in-kind delivery of subsystems need to be recognised
- Powerful remote access networks
- Remote access security





Limitations of existing Control Systems: JET as an example

High Maintenance

- Proliferation of interfaces
- •Should converge on modern instrumentation, computer, etc. standards

Lack of commonality and functionality between different devices RTMC systems

•e.g. JET PPCC not simply exportable to other devices and viceversa

Lack of flexibility

•Integration of new equipment and physics into the existing infrastructure is time-consuming.

Lack of good transport and integrated models and tools

•Integrated development environment and interchange formats

Future developments should acknowledge these issues

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The challenge: Control & Data Acquisition in Long Discharges

When the duration of the fusion plasmas increases, it is not convenient to acquire all data provided by all diagnostics

Therefore:

(i) Diagnostics are divided in two groups: technical and scientific;

(ii) Technical diagnostics are in continuous operation, connected to the data acquisition system (DAS) running in real-time algorithms for data analysis and control;

(iii) Scientific diagnostics driven by detection of an event. The DAS generates a signal that is distributed to all transient recorders to adequate their operation to study the detected event. The aim is data compression/reduction.

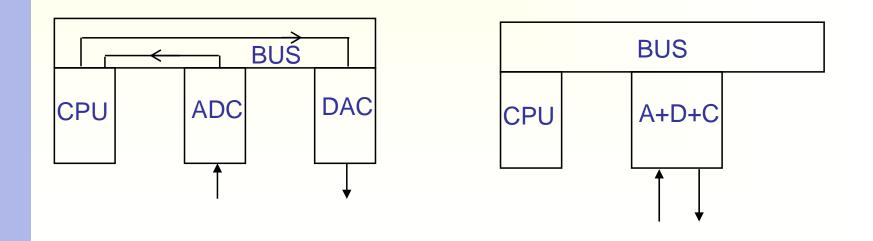
(iv) The transient recorders must have pre-trigger capabilities, aiming at not to loose any information



Traditional Architecture

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- Control and data acquisition processes usually run on separate hardware platforms
- Data links for exchange of relevant data and events not always provide the bandwidth and low-latency adequate for real-time operation
- The large amount of raw data burdens the data links and the software based signal processing units



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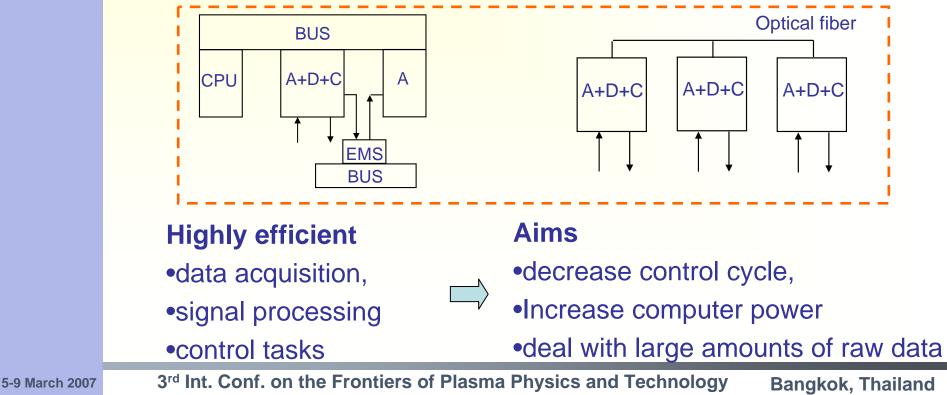
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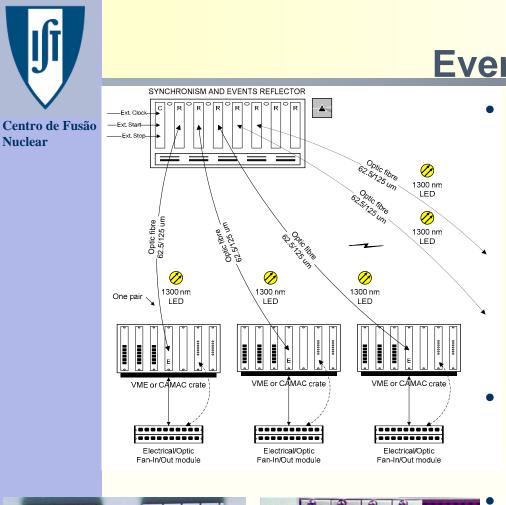


New Architectural Paradigms

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- New Real-time control systems are based on intelligent modules
- Control loop simple algorithm performed at the data acquisition level
 - reduced data rate flow output
 - Low latency data link transmits reduced data to software signal processors and control signals back to the actuator units
 - Helps attaining low loop delays
- Integrated support for synchronization of processes an time is desirable







MAST Fast Timing and Event Management System

- Main features
 - Broadcasts timing and peer-topeer events
 - Overall timing precision of the system is 1 µs
 - Events propagated in less than 2 µs
 - Up to 64k global distinct events
 - Up to 17 Reflector units and 256 EPN modules provide up to 2048 I/O channels

REFLECTOR unit

 Links up to 16 EPN (128 I/O channels) up to distances of 300 m through optic fiber at 1 GBit rate

EPN module

- Parallel Processing, DSP based, VME Module
- 8 output timing unit; up to 10 MHz complex pulse patterns.
- 8 input programmable event generator.

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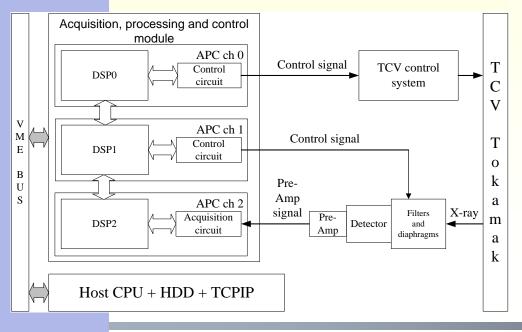
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Real-Time Pulse Height Analysis X-Ray Diagnostic

TCV Diagnostic composed by detector, amplifiers and data acquisition system (multichannel analyser or transient recorder + software for data analysis).

Off-line capabilities:

- Determination of average plasma electron temperature
 - a) during the discharge (single value)
 - b) in several periods of the discharge (increasing the discharge duration and X-ray intensity).
- Identification of plasma impurities.



CFN's intelligent module allowed:

- Real-time measurements each 100 ms (300 kcounts/s)
- **Pile-ups avoidance** (realtime adjustment of diaphragm between plasma and detector).
- Real-time control of the gyrotrons used for plasma auxiliary heating

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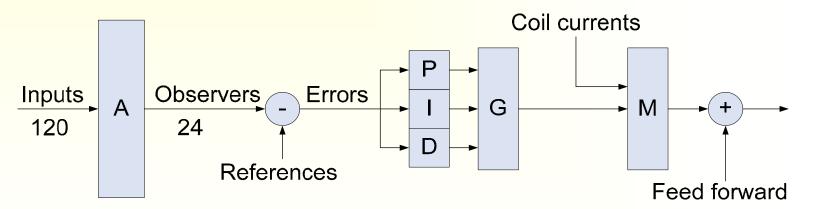
TCV Advanced Plasma Control System

Centro de Fusão Nuclear TCV plasma control system had some limitations since was mainly based on analogue components:

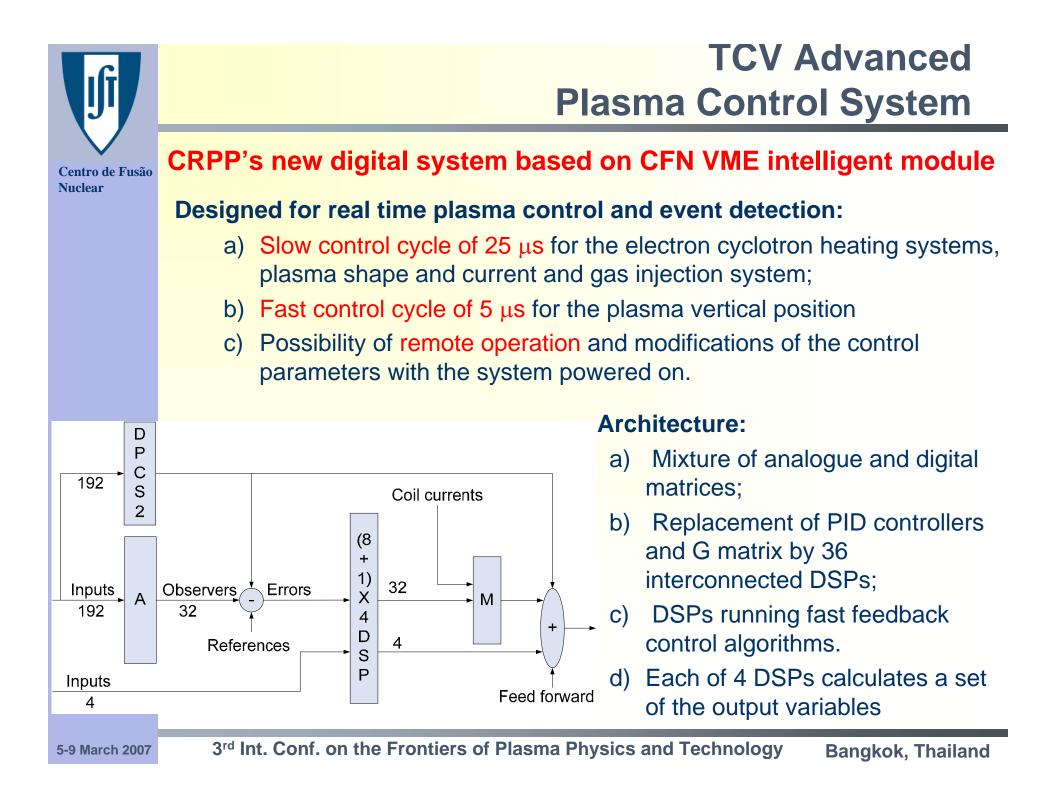
(i) A, G and M matrices are based on analogue I/O, with digital coefficients provide by fixed gain PID (Programmable Integral Differentiator) controllers;

(ii) No real-time programming capability;

(iii) Only linear control algorithms could be implemented.



Reducing these limitations was particularly important to enhance the exploitation of a device designed for the study of plasma control in different plasma configurations.



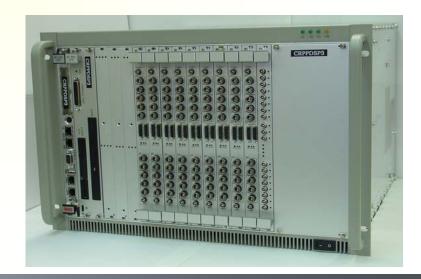


TCV Advanced Plasma Control System

 The slow control has been implemented by 32 DSPs in 8 modules.

- The fast control has been implemented in 4 DSPs of one module
- Specially developed for this application:
 - -1 Extra digital inputs and outputs (XIO) board.
 - -1 DataMover bus (DMBUS) board.

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VME host	HDD+F	Connector P2 MI1	Connector P2 MI2	Connector P2 MI3	Connector P2 MI4	Connector P2 MI5	Connector P2 MI6	[Connector P2] MI7	Connector P2 MI8	Connector P2 MI9 (fast ctl)	B Connector P2 XIO



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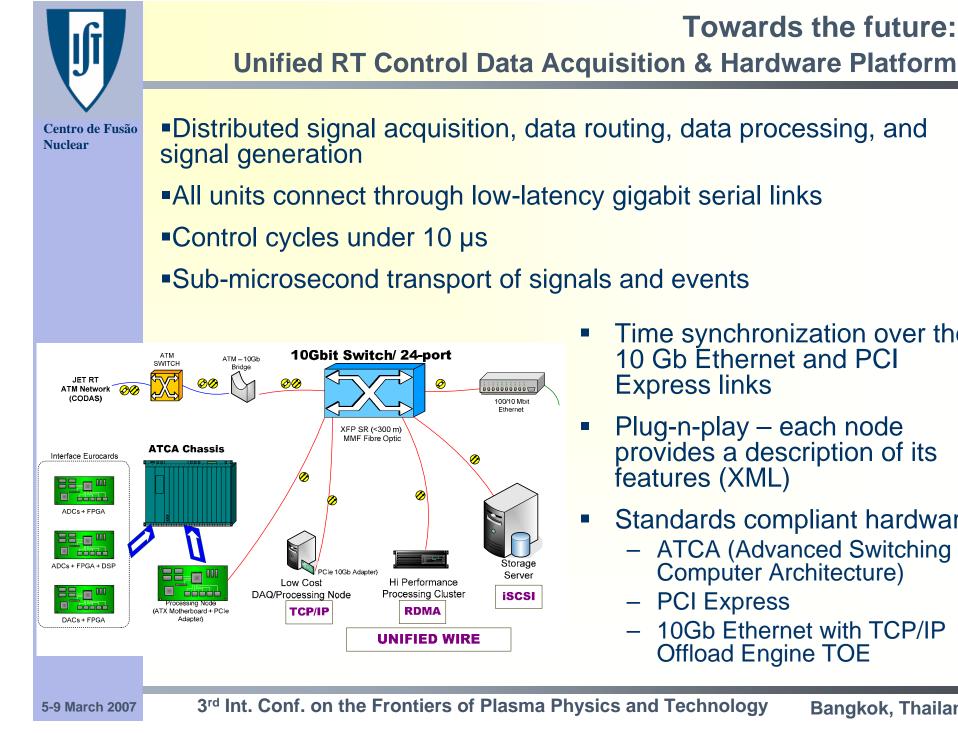
Real-Time Control Systems Test-Bench for JET

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- Tests the operation of a real-time control system/tool:
 - Stimulates the inputs with the previously stored plasma variables or user defined waveforms
 - Simultaneously records the output variables



- Non-intrusive test method
 - Does not harm the machine to be controlled by unstable on-test control systems;
 - Permits to test all possible operation scenarios;
 - Allows faster test duty-cycles.
- First version of the system implemented in PCI:
 - 32 channels analogue stimulus generator;
 - 8 channels transient recorder;
 - Signals and parameters of previous experiments retrieved from a MDSplus server;



Time synchronization over the 10 Gb Ethernet and PCI **Express** links

Towards the future:

Plug-n-play – each node provides a description of its features (XML)

- Standards compliant hardware
 - ATCA (Advanced Switching Computer Architecture)
 - PCI Express
 - 10Gb Ethernet with TCP/IP Offload Engine TOE

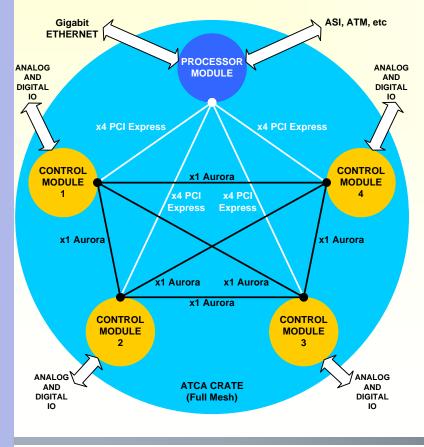
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JET Vertical Stabilisation Upgrade

Vertical Stabilization Controller (input signals > 50):

- Centro de Fusão x86-based ATCA controller
 - up to 12 DGP cards (PCIe links through the ATCA full mesh backplane)
 - parallel execution on FPGAs for MIMO signal processing (Control loop delay < 50 μ s, aim < 10 μ s)



Specifications:

- Reduce delays on acquisition/generator endpoints and data interconnect links.
- High processing power near acquisition/generator endpoints and system controller.
- Synchronism of all digitizer/generator endpoints.
- Maintainability, upgradeability and scalability.
- Low cost per channel.
- Low risk of implementation and testing.



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FireSignal – Data Acquisition and Control System Software

- Highly generic Control and Data Acquisition System
- Provides a standard for hardware description
- Integration of new hardware in a plug and play, XML based, philosophy
- Remote hardware configuration through intuitive forms
- Easy access to acquired data by authorized personal
- Remote participation from anywhere in the world
- Personalized data viewers for different data types through a plug-in system.
- Data sharing between users
- Integration of any kind of database
- Implements a generic security access schema



FireSignal – Data Acquisition and Control System Software

Successfully used at ISTTOK, ETE (Brazil) and CASTOR. Under tests at TCA/Br

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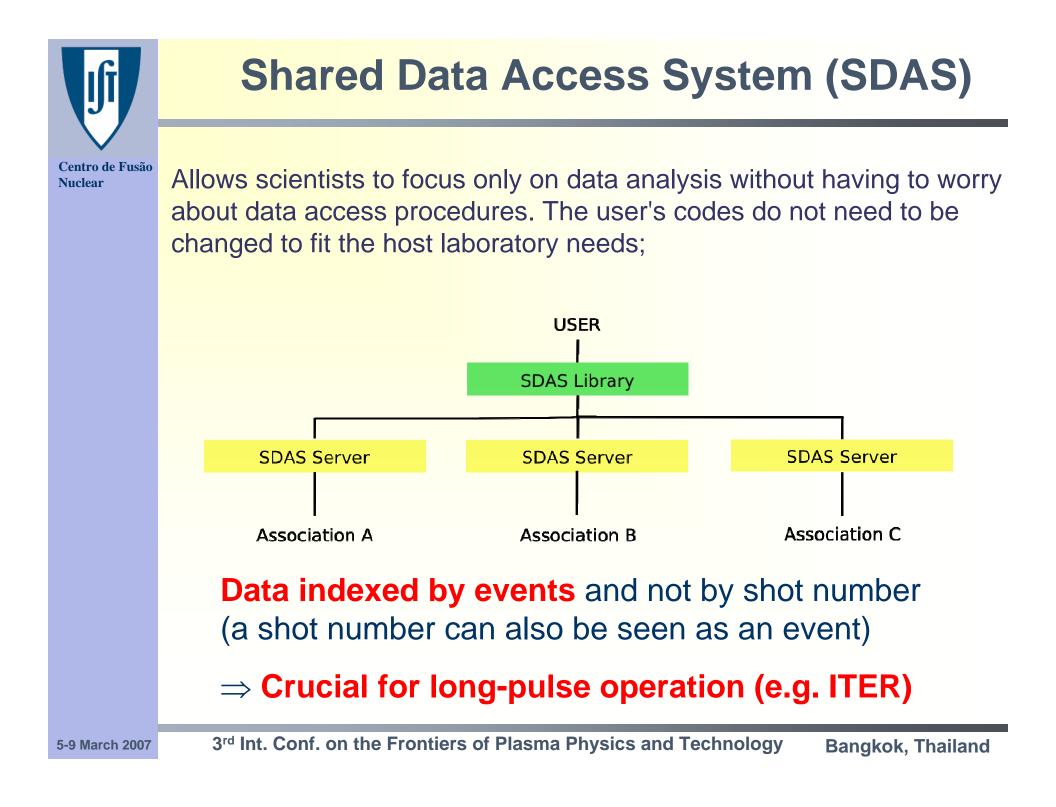
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Shared Data Access System (SDAS)

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- Provides a valid alternative for data indexing. Using events and time-stamps the system is prepared for the challenges of the near future, maintaining backward compatibility with the solutions of the past.
- SDAS libraries are easily integrated in programs such as MatLab, Mathematica and IDL, allowing users to retrieve data with only a few lines of code;
- Each experiment has a SDAS server where the software layer runs;
- A software layer provides the same data accessing functions in all experiments;
- The server communicates and translates the queries to the experiment's data ;
- The system was designed to be as generic as possible and can be adapted to any data storage mechanism.





The way forward: Advanced Control system for COMPASS-D

The development of a control and data acquisition system for COMPASS-D represents an opportunity to test ITER relevant solutions

The following areas are relevant for testing at COMPASS-D as an ITER model

- Generic Plant Systems
- Self-description of Plant Systems
- Supervisory Control System, automation and remote requests
- Pulse schedule, editing and validation
- Data naming and data access

CFN developments address already some of these needs

CFN participation will build on previous developments towards ITER relevant solutions



Centro de Fusão Nuclear ITER Control and data acquisition will be challenging

Advanced Real-time control systems will be necessary and have to be timely developed

CFN has been actively involved in developing innovative control and data acquisition systems for several fusion devices

Some CFN developments approach ITER needs

CFN will extensively test the remote access tools in the 3rd Joint Experiment in ISTTOK organized in collaboration with IAEA (October 2007)

New machines (or enhancements of the existent) represent crucial opportunities to explore new concepts compatible with ITER requirements

CFN's future work will build on previous developments towards ITER relevant solutions



CFN (Association EURATOM/IST): a meaningful contribution

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- Advanced Plasma Control Systems
- Fast Timing and Event Management System
- Shared Data Access System
- Generic Control and Data Acquisition
 System
- Hybrid (transient recorders and waveform generators) intelligent modules for CAMAC, VME, PCI, PCIe and ATCA standards;
- Unified RT Control Data Acquisition & Hardware Platform
- New concepts of data transmission and networks
- Remote operation
- Real-Time Control Systems Test-Bench
- Parallel processing techniques

Integration into small, medium and large fusion devices



Broader outreach by establishing new links with different plasma physics and fusion communities benefiting from IAEA activities