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Cooled Reactors in the 21st Century
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***Open Issues Associated with Passive Safety
Systems Reliability Assessment***

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Outline

- **Introduction**
 - **Passive Systems**
 - **Passive Systems Reliability and Safety**
 - **Applications to advanced reactors**
 - **Thermal-hydraulic (t-h) Passive Systems**
- **Reliability Assessment Approaches**
- **Open Issues and Implementation**
 - **Uncertainties**
 - **Dependencies**
 - **Integration into accident sequences** within a psa framework
 - **Passive vs active systems**
- **Summary**
- **Outlook**

Generics

- **Innovative** reactors largely implement **passive** safety systems
- Reactivity control, decay heat removal, fission product containment
- Applications of passive systems for innovative reactors demand high **availability** and **reliability**
- **PSA** analysis
- Accident sequence definition and assessment
 - **Event Tree and Fault Tree model**
- Introduction of a passive system within an accident scenario in the fashion of a **front-line system** and in combination with active systems and human actions

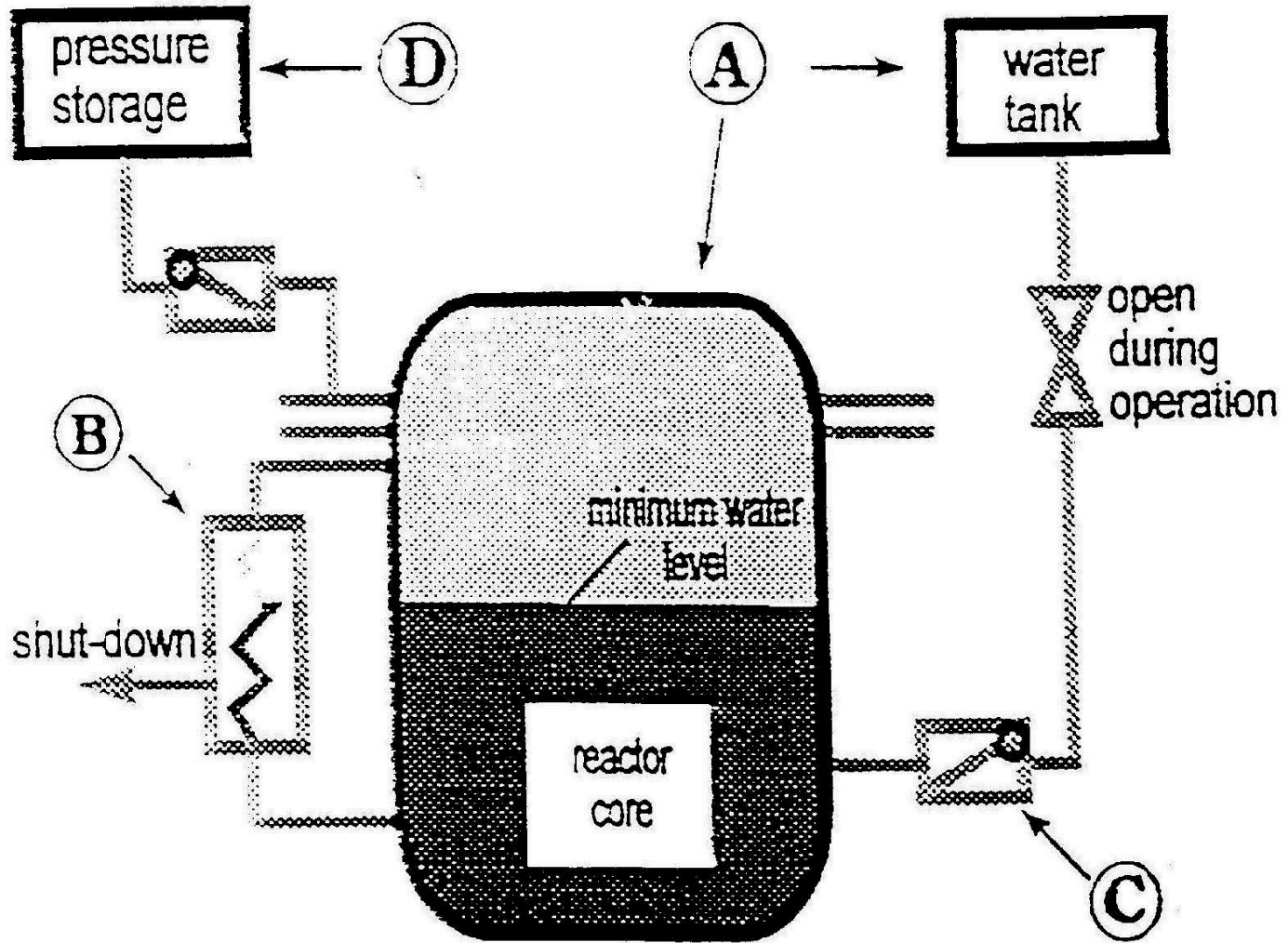
Recalls

- **IAEA (IAEA-TECDOC-626) definitions:**
 - ***Passive Component***: a component which does not need any external input to operate
 - ***Passive System***: either a system which is composed entirely of passive components and structures or a system which uses active components in a very limited way to initiate subsequent passive operation
- **Passive System Categorization:**
 - **A:** physical barriers and static structures,
 - **B:** **moving working fluids,**
 - **C:** moving mechanical parts,
 - **D:** external signals and stored energy (passive execution/active initiation)

Classification of Passive Systems

	Category-A	Category-B	Category-C	Category-D
Input Signal, External Power Sources, Forces	No	No	No	Yes
Moving Mechanical Parts	No	No	Yes	Yes
Moving Working Fluid	No	Yes	Yes/No	Yes/No
Some examples	<ul style="list-style-type: none"> ■ Core cooling system relying only on radiation/conduction ■ Physical barriers against release of fission products 	<ul style="list-style-type: none"> ■ Reactor cooling based on natural circulation 	<ul style="list-style-type: none"> ■ Systems consisting of accumulators or storage tanks and discharge lines equipped with check valves. ■ Mechanical actuators such as check valves and spring loaded relief valves 	<ul style="list-style-type: none"> ■ Emergency core cooling systems based on gravity/compressed Nitrogen driven flow of water activated by battery-powered valves. ■ Mechanical Shut-Off rods

Examples



Passive Systems Reliability

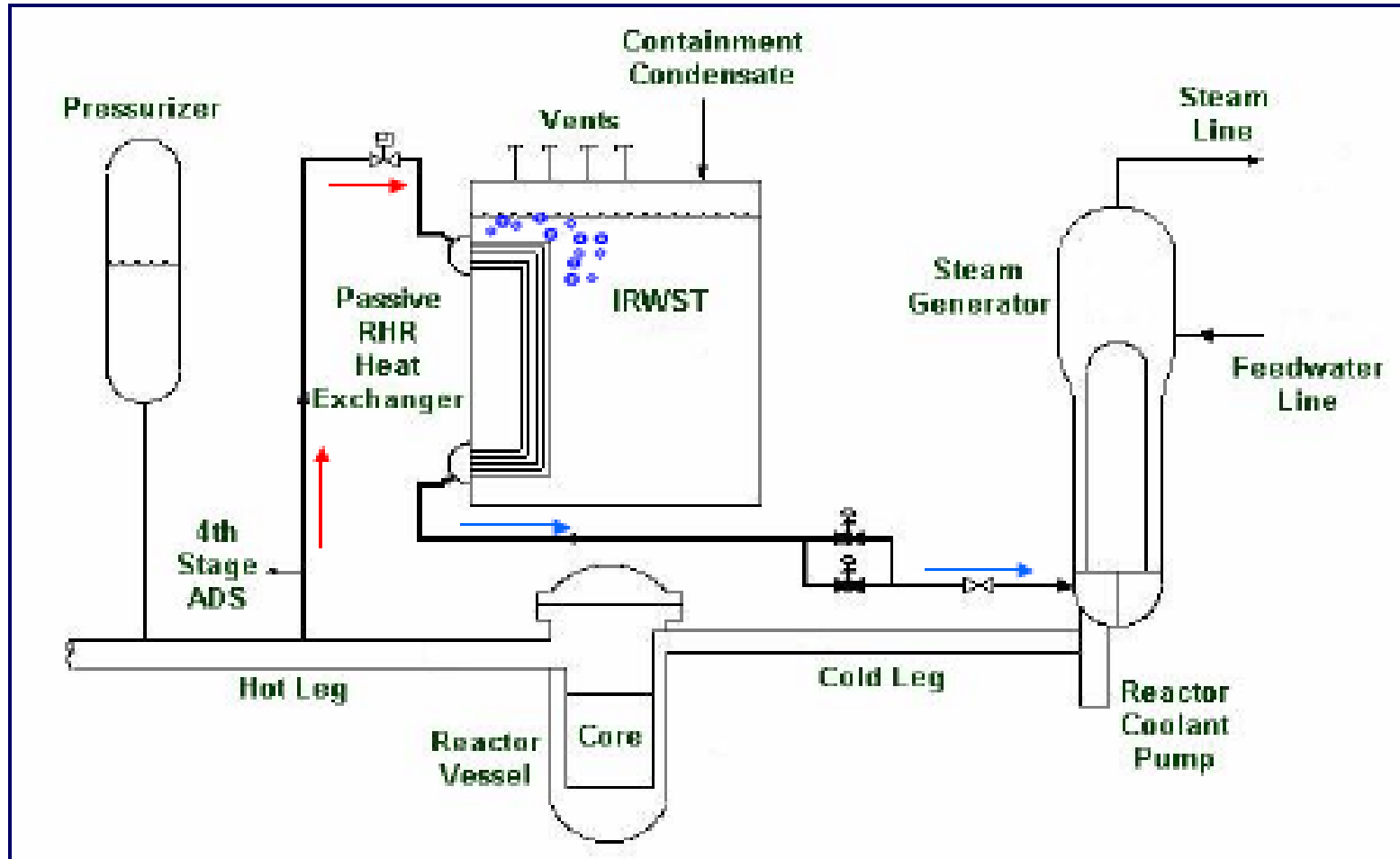
- Probabilistic reliability methods for passive **A** safety functions have been extensively developed and applied in **fracture mechanics**
- For several passive **C** and **D** systems reliability figures may be derived from **operating experience**
- For **passive B** type systems basing on **physical principle (natural circulation)** denoted as **t-h (thermal-hydraulic)** passive systems, there is **no agreed** approach towards their reliability assessment yet
 - **Deviations** of natural forces or physical principles from the **expected** conditions, rather than classical component mechanical and electrical faults
 - **System/component reliability** (piping, valves, etc.)
 - mechanical component reliability
 - **Physical phenomena “stability”** (natural circulation)
 - **factors impairing** the **performance/stability** of the physical principle (buoyancy and density difference) upon which passive system operation is relying
- NEA CSNI/WGRISK Workshop on *Passive Systems Reliability—A Challenge to Reliability, Engineering and Licensing of Advanced Nuclear Power Plants*, Cadarache, (F), 4-6/03/'02, NEA/CSNI/R(2002)10
- IAEA-TECDOC-1474, *Natural circulation in water cooled nuclear power plants. Phenomena, models, and methodology for system reliability assessments*, 2005

Thermal-hydraulic Passive Systems

- **Natural circulation:** small engaged driving forces and thermal-hydraulic factors affecting the passive system performance (e.g. non condensable fraction, heat losses)
- System from the **predictable** nominal performance to the state of degradation of the physical principle in varying degrees up to the failure
- Occurrence of **physical phenomena** leading to pertinent failure modes
- Physical principle deterioration dependency on the **boundary conditions** and **mechanisms** needed for start-up and maintain the **intrinsic** principle
- Passive Systems for **decay heat removal** implementing in-pool heat exchangers and foreseeing the free convection (e.g. **PRHR** for AP 600 and AP 1000, **Isolation Condenser** for SBWR and ESBWR)

T-h Passive Systems in Advanced reactors

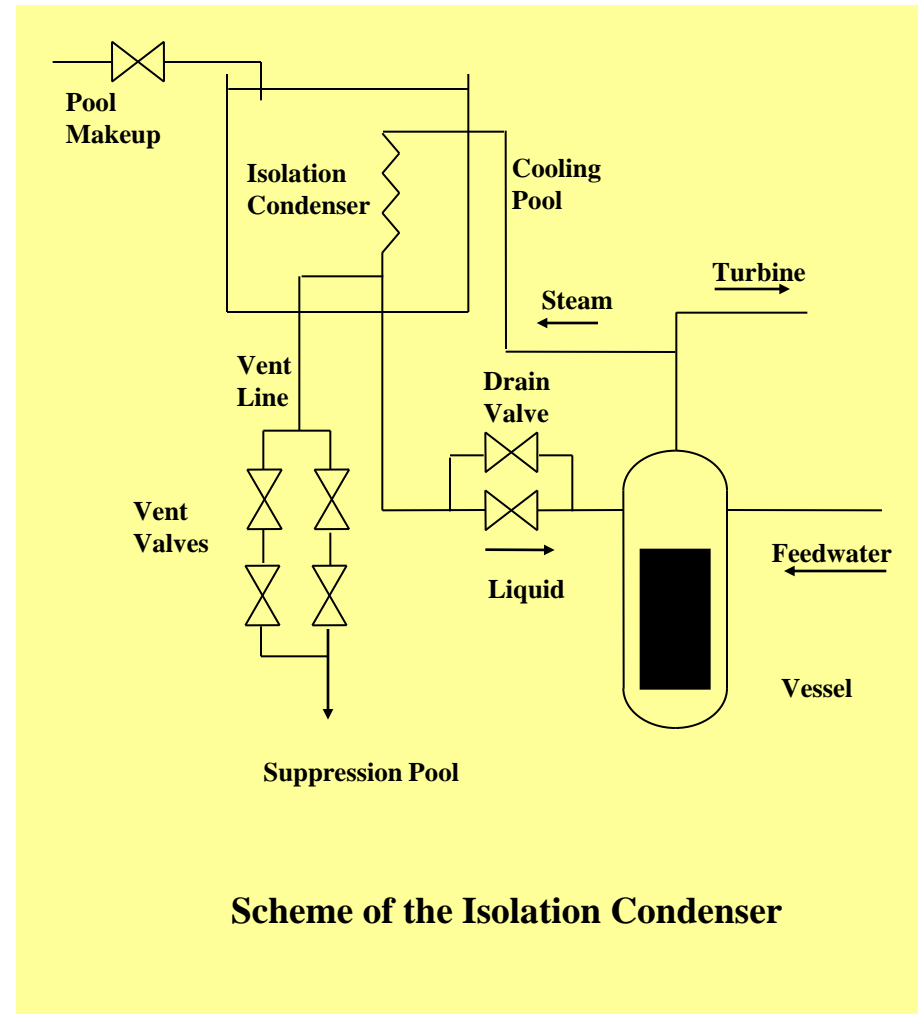
AP600/AP1000 Passive Residual Heat Removal (PRHR) System



T-h Passive Systems in Advanced reactors

Isolation Condenser (SBWR, ESBWR)

- Core Decay Heat removal from the reactor, by natural circulation following an isolation transient, including a **heat source** and a **heat sink** where **condensation** occurs via a heat exchanger
- Limit the **overpressure** in the reactor system at a value below the set-point of the safety relief valves, preventing unnecessary reactor depressurization
- Isolation Condenser **actuation** on MSIV position, high reactor pressure and low reactor level



Reliability Assessment Approaches (basics)

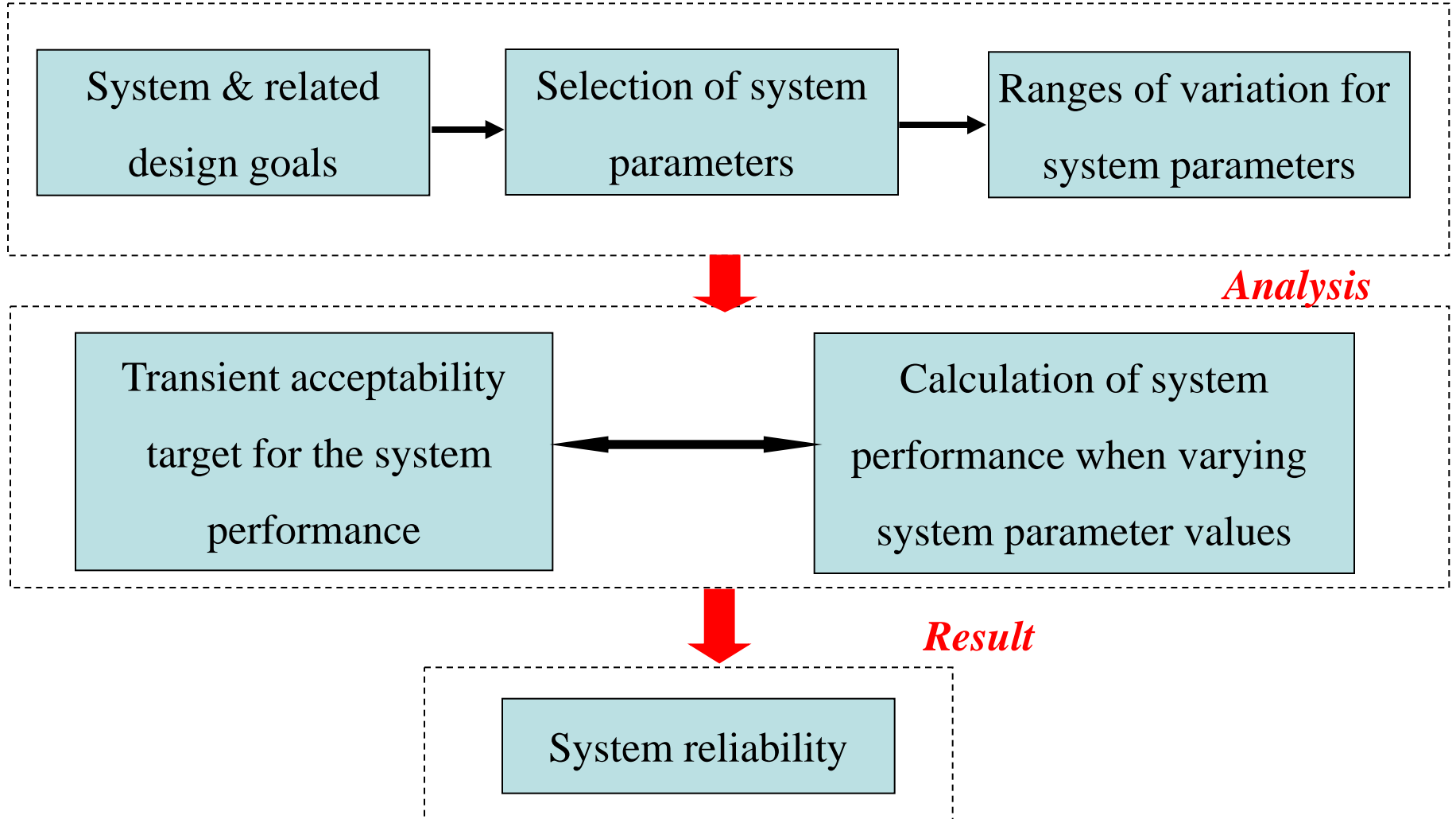
- To provide **essentials** for passive system reliability assessment (**ENEA**)
- Approach based on **independent failure modes**
 - Burgazzi L., *Evaluation of Uncertainties Related to Passive Systems Performance*, Nuclear Engineering and Design, Volume 230, May 2004, pp-93-106
 - Burgazzi L., *Addressing the Uncertainties Related to Passive System Reliability*, Progress in Nuclear Energy, Vol. 49, pp. 93-102, January 2007
- Approach based on **failure modes** of passive system **hardware** components
 - Burgazzi L., *Passive System Reliability Analysis: a Study on the Isolation Condenser*, Nuclear Technology, Vol. 139, pp. 3-9, July 2002
 - Burgazzi L., *Failure Mode and Effect Analysis for the Safety and Reliability Analysis of a Passive System*, Nuclear Technology, Vol. 156, pp.150-158, November 2006
- **Functional reliability** or **load-capacity** approach
 - Burgazzi L., *Reliability Evaluation of Passive System through Functional Reliability Assessment*, Nuclear Technology, Volume 144, pp. 145-151, November 2003
 - Burgazzi L., *Thermal-hydraulic Passive System Reliability-Based Approach*, Reliability Engineering and System Safety, Vol. 92, pp. 1250-1257, September 2007

Reliability Assessment Approches (integrated methods)

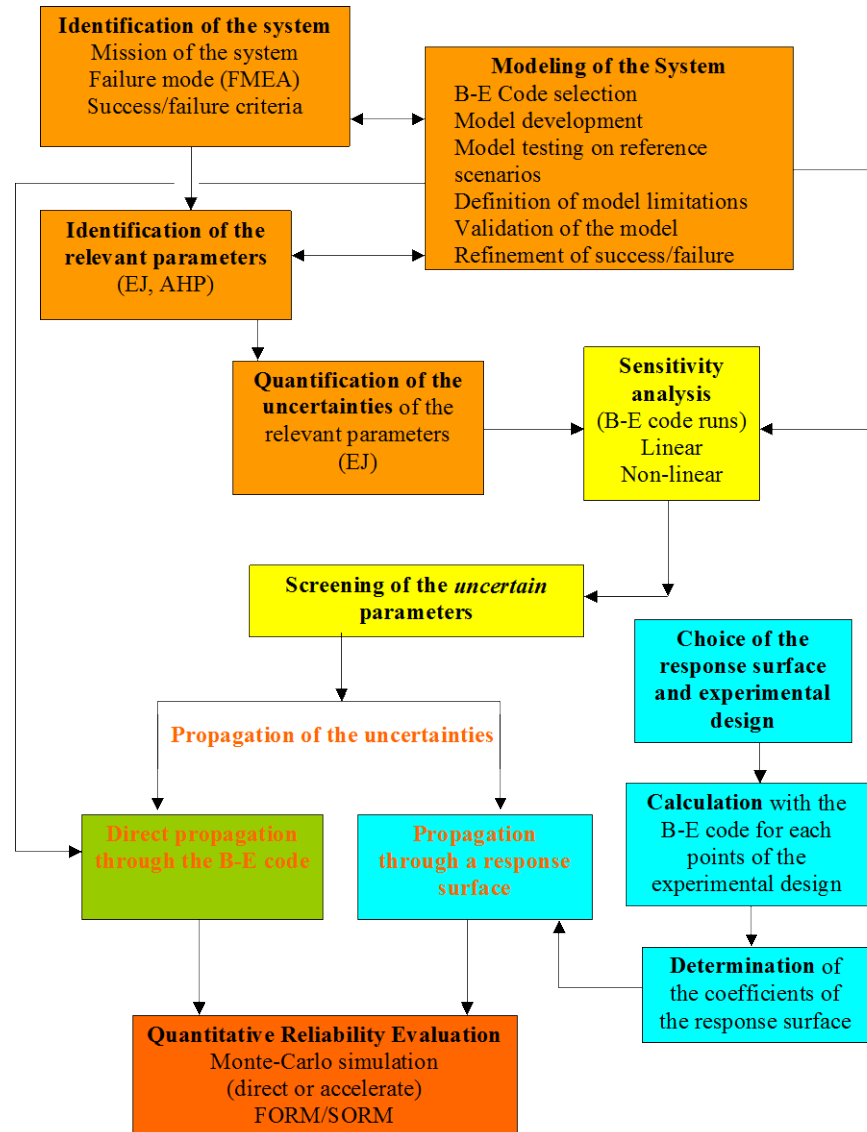
- To achieve a more **consistent** methodology,
 - to include t-h code simulations
 - to capture all the phenomena involved and their interactions
 - to merge **probabilistic** and **physical**, i.e. t-h, aspects
- **REPAS (REliability of PAssive Systems)**
 - ENEA, University of Pisa, Polytechnic of Milano, University of Rome
 - J. Jafari, F.D'Auria, H. Kazeminejd, H. Davilu, *Reliability evaluation of a natural circulation system*, *Nuclear Engineering and Design* 224 (2003) 79–104
- **RMPS (REliability Methods for PAssive Safety Functions)**
 - Fifth European Union Framework Programme project (2001-2004)
 - Marques M., et al., *Methodology for the reliability evaluation of a passive system and its integration into a Probabilistic Safety Assessment*, *Nuclear Engineering and Design* 235 (2005) 2612–2631
- **APSRA (Assessment of PAssive System ReliAbility)**
 - Bhabha Atomic Research Centre (India)
 - Nayak A. K., et al., *Passive system reliability analysis using the APSRA methodology*, *Nuclear Engineering and Design*, Volume: 238, Issue: 6, June, 2008, pp. 1430-1440
 - Nayak A.K et al., *Reliability assessment of passive isolation condenser system of AHWR using APSRA methodology*, *Reliability Engineering and System Safety*, Volume: 94, Issue: 6, June, 2009, pp. 1064-1075

REPAS Method

Simplified diagram of the REPAS methodology



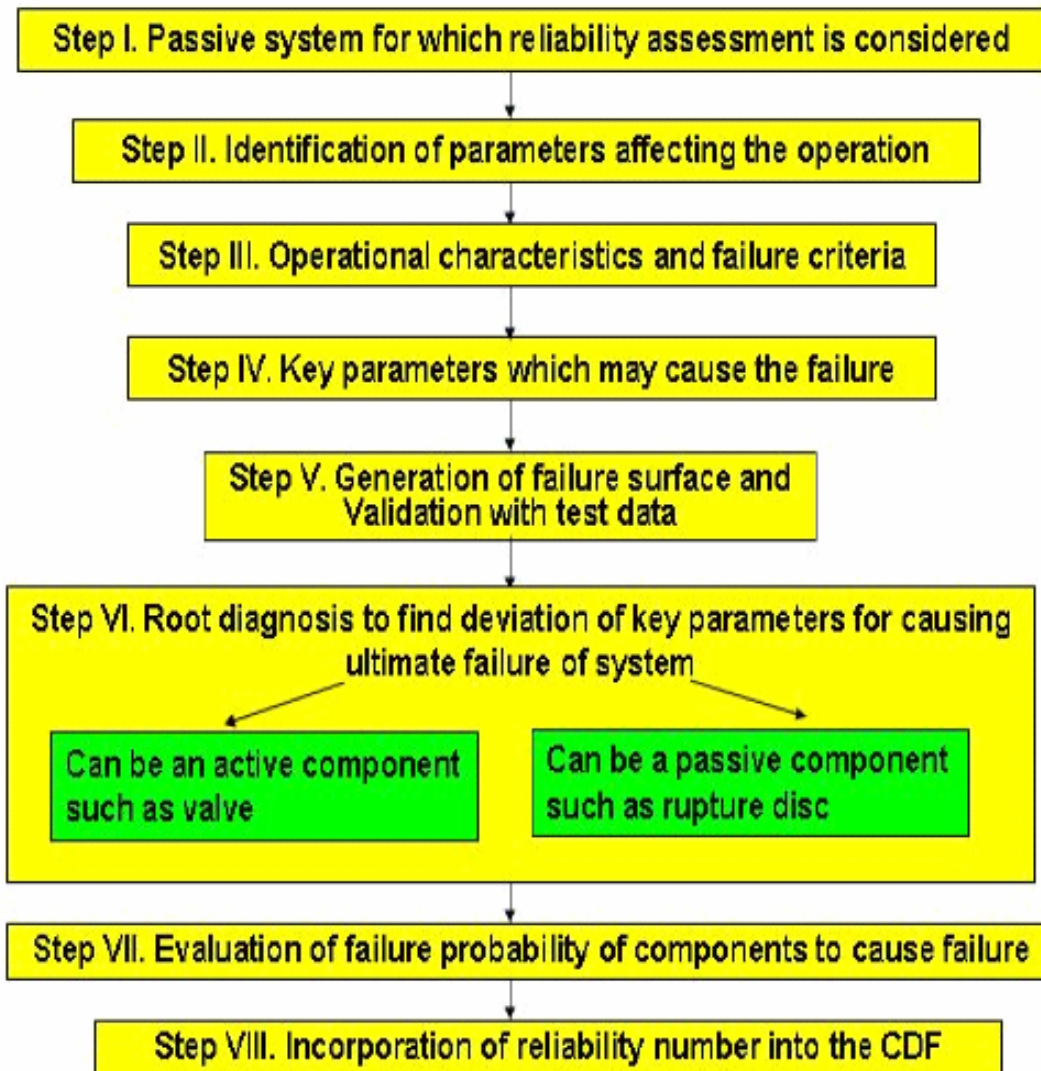
RMPS Methodology: roadmap



RMPS Methodology: Objectives

- To propose a **specific** methodology to evaluate the reliability of passive systems
- Identification and quantification of the sources of **uncertainties** and determination of the **important** variables
- **Propagation** of the uncertainties through a T-H model and **reliability** evaluation of the T-H passive system
- **Integration** of the T-H passive system in an accident sequence, as a basic event
- **www.rmmps.info**

APSRA Methodology



Assessment of Passive Systems Reliability (APSRA)

- **Failure surface**
- **Deviations** of all critical parameters influencing the system performance
- Causes of deviation through **root** diagnosis
- **Mechanical** components (as valves, control systems, etc.)
- **Failure probability** through **classical PSA** (fault tree)
- **Comparison** of test data with t-h code prediction to reduce uncertainties

Open issues related to t-h passive systems reliability

- **Analysis** of the different methodologies proposed so far
- **Uncertainties**
 - Passive system performance
 - T-h code
- **Dependencies**
 - Relevant variables
- **Integration** of passive systems into an **accident sequence** within a psa framework
- Passive vs **active** systems

Sources of uncertainties related to passive system performance

- **Uncertainties** related to natural circulation system behaviour prediction
 - **Deviations** of the natural forces or physical principles from the expected conditions
 - **Phenomenological uncertainties**, due to **scarcity** of operational and experimental data
 - **Epistemic Uncertainties**, i.e. related to the state of knowledge
- **Difficulties** in performing meaningful reliability analysis and deriving credible reliability figures
 - Expert judgment elicitation and engineering/subjective judgment
- Burgazzi L., *Evaluation of Uncertainties Related to Passive Systems Performance*, Nuclear Engineering and Design Volume 230, May 2004, pp 93-106

Aleatory

Geometrical properties

Material properties

Initial/boundary conditions (design parameters)

Epistemic

T-H analysis

Model (correlations)

Parameters

System failure analysis

Failure criteria

Failure modes (critical parameters)

Categories of uncertainties associated with T-H passive systems reliability assessment

Zio, E., Pedroni, N., *Building confidence in the reliability assessment of thermal hydraulic passive systems. Reliability Engineering and System Safety*, 94 (2009), 268-281

Sources of uncertainties related to t-h code

- Uncertainties in the **best estimate codes** can arise due to e.g.,
 - Inadequate **physical models** built in the codes to represent a specific phenomena;
 - Absence of **models** to represent a particular phenomena;
 - Approximation in simulating system **geometry**;
 - Deviations of the input parameters in respect of **initial** and **boundary conditions**;
 - Uncertainties in **thermophysical properties** and **thermohydraulic relationships**.
- The **uncertainty** analysis (of a code prediction) implies a procedure to evaluate the precision (or the error) that characterizes the application of a best-estimate code
- The **reliability** analysis (of a system) aims at characterizing the ability of a system '*to operate satisfactorily*', following assigned specifications, over a period of time
- Therefore the **uncertainty** of the code can affect the **prediction** of the system

Uncertainty and sensitivity qualitative analysis

Grade Rank for Uncertainty and Sensitivity

	Grade	Definition
Uncertainty	H	The phenomenon is not represented in the computer modelling or the model is too complex or inappropriate which indicates that the calculation results will have a high degree of uncertainty.
	M	The phenomenon is represented by simple modelling based on experimental observations or results.
	L	The phenomenon is modelled in a detailed way with adequate validation.
Sensitivity	H	The phenomenon is expected to have a significant impact on the system failure
	M	The phenomenon is expected to have a moderate impact on the system failure
	L	The phenomenon is expected to have only a small impact on the system failure

Failure Modes related Uncertainty and Sensitivity

TOPIC	UNCERTAINTY	SENSITIVITY
Envelope failure	L	H
Cracking	L	L
Non-condensable gas	H	H
Thermal stratification	H	H
Surface modification	M	L

Burgazzi L., *Evaluation of Uncertainties Related to Passive Systems Performance*, Nuclear Engineering and Design Volume 230, May 2004, pp 93-106

Expert judgment elicitation process

Open Issues: Dependencies

- Assumption of **independence** among relevant parameters adopted in the analysis (zero covariance)
 - safety variables
 - e.g. flow rate, exchanged heat
 - critical parameters driving the modes of failure
 - e.g. non-condensable gas
- In case of **dependence (e.g. degradation measures)**, parameters can not be combined freely and independently
- **Joint pdfs**, e.g. multivariate distributions
- **Conditional** subjective probability distributions
- **Covariance** matrix
- **Functional** relationships between the parameters
- Burgazzi L., *Reliability Prediction of Passive Systems based on Bivariate Probability Distributions*, Nuclear Technology, Volume 161, pp. 1-7, January 2008
- Burgazzi, L., *Evaluation of the Dependencies Related to Passive System Failure*, accepted for publication in Nuclear Engineering and Design, DOI information: <http://dx.doi.org/10.1016/j.nucengdes.2009.08.019>

Open Issues: Integration of passive systems within an accident sequence

- Limitations of PSA (**event tree** development)
 - **Binary** representation (success or failure, **intermediate** states are usually not treated)
 - Time treatment (chronology of events instead of actual timing)
- Need for the development of **dynamic event tree** in order to evaluate the interaction between the **parameter evolution** during the accident and the system state
- Evaluation for **72** hours grace period, compared to 24 hrs in classical PSA
- **Time-variant stochastic** process
 - the evolution of physical parameters over time, in terms of probability distributions
- Burgazzi, L., *About Time-variant Reliability Analysis with reference to Passive Systems Assessment*, Reliability Engineering and System Safety, Vol. 93, pp.1682-1688, 2008

Open Issues: Active vs Passive

- **Functional** and **economic** comparison of active vs passive safety systems, required to accomplish the same **mission**
- **Passive**
 - Advantages e.g.,
 - No **external power supply**: no loss of power accident
 - No **human factor**
 - Better **impact** on public acceptance, due to the presence of “natural forces”
 - Less complex system than active and therefore economic competitiveness
 - Drawbacks e.g.,
 - Reliance on “**low driving forces**”, as a source of uncertainty
 - **Licensing** requirement (open issue)
 - Reliability assessemnt in any case (lack of data)

Conclusions and Path forward (1/3)

- As the future reactor concept makes use of **passive safety features** in combination with active safety systems, the question of Natural Circulation Decay Heat Removal (NCDHR) reliability and performance assessment into the ongoing PSA constitutes a **challenge**
- Development of a **consistent methodology** for the evaluation of the reliability of the passive systems
- **Future needs**
 - *Clear rules for identification and quantification of uncertainties.*
 - *Formal expert judgment (EJ) protocol to estimate distributions for parameters whose values are either sparse or not available*
 - *Sensitivity analysis techniques to estimate the impact of changes in the input parameter distributions on the reliability estimates*
 - *Clear distinction between the prediction of the thermal hydraulic code and the true behaviour of the passive system under consideration.*
 - *Problem of model uncertainties*
 - *The time dependence of the passive system reliability*
 - *Dynamic event trees*

Conclusions and Path forward (2/3)

Future needs (following):

- Evaluation of the *dependencies* among relevant system parameters
- Comparison of *different methodologies*
- *Merge* elements of different methodologies : RMPS, APSRA/BARC, REPAS and ENEA methodologies, since high *dependency* of results upon the assumptions underlying the models
- Establish *guidelines* and *criteria* for the comparison of active and passive systems

Conclusions and Path forward (3/3)

International efforts in progress

- **IAEA Coordinated research project (CRP) on “*Development of Methodologies for the Assessment of Passive Safety System Performance in Advanced Reactors*” (2008-2011)**
 - the objective is to determine a **common analysis-and-test method** for reliability assessment of passive safety system performance
- **IAEA CRP on “*Natural Circulation Phenomena, Modelling and Reliability of Passive Systems*” (2004-2008)**
 - **TECDOC-1474, “Natural Circulation in Water Cooled Nuclear Power Plants”, November 2005**
 - **TECDOC-XXXX, “Passive Safety Systems and Natural Circulation in Water Cooled Nuclear Power Plants”, ready for publication**
 - **TECDOC-XXXX, “Natural Circulation in Water-Cooled Nuclear Power Plants: Phenomena, Modelling, and Reliability of Passive Systems that Utilize Natural Circulation”, under preparation**