AP1000: The PWR Revisited

IAEA
International Conference on Opportunities and Challenges for Water Cooled Reactors in the 21st Century

Paolo Gaio
Westinghouse Electric Company
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Background

- Late ’80: USA Utilities under direction of EPRI and endorsed by NRC: *Advanced Light Water Reactor Utility Requirements Document (URD)* with policy and design requirements for next generation
- URD addresses **evolutionary** and **passive** LWR
- **Passive** has much higher expectations (ex. maintain safe shutdown for 72 hrs. after design base event w/o operator action vs. 30’ for evolutionary)
- **Passive** is also *simpler, smaller and much improved*
- In Europe similar document *European Utility Requirements (EUR)*
AP 1000: The Reactor Coolant System
Familiar but Improved Reactor Coolant System

- Two loop plant: two cold and one hot leg
- Four Reactor coolant pumps mounted in steam generator lower head - No RCP shaft seals
- Larger pressurizer (2100 ft - 50% larger than operating plants) eliminate PORV
- Top-mounted, fixed in-core detectors
- Primary pipes forged in one piece reduces welds 50%, supports 80%
- Ring-forged reactor vessel (no longitudinal welds)
- All-welded core shroud (not bolted)
- Fuel, Internals, Reactor Vessel
  - Similar to Doel 4, Tihange 3, S. Texas
  - Improved materials - 60 yr life
- Steam Generators
  - Similar to large Westinghouse SGs in operation
AP1000 Reactor Vessel

- Westinghouse 3-loop Reactor
  - 3.99m (159in) ID, 157 fuel assemblies
  - *Longer vessel to accommodate 4.27m (14ft) fuel*
  - Ring forged construction
    - No welds in core region
  - Improved materials permit 60-yr design life
  - *W-CE-type core shroud*
    - Replaces radial reflector
    - All-welded design
  - Top-mounted in-core I&C
    - Fixed position, online readout
    - No penetrations in bottom vessel
AP1000 Steam Generator

- Based on proven Westinghouse designs
  - AP1000 design based on ANO RSG
  - Design Features
    - Inconel 690 TT tubes
    - Stainless steel support plates
    - Improved access
    - Excellent operating experience
  - Over 1,200 SG years of operation
  - Less than 0.1% total tubes plugged
AP1000 High Efficiency Reactor Coolant Pump

• Canned Motor Pump
  - No shaft seal leakage or support system
  - No seal (LOCA benefit)
  - No oil cooling system
  - Reduced maintenance
  - Excellent operating experience
  - High inertia bearing
  - Pump design will be verified through extensive testing
## Comparison of Selected Parameters

<table>
<thead>
<tr>
<th>PARAMETER</th>
<th>Doel 4 / Tihange 3</th>
<th>AP1000</th>
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</thead>
<tbody>
<tr>
<td>Net Electric Output, MWe(2.5&quot;HgA)</td>
<td>985</td>
<td>1117</td>
</tr>
<tr>
<td>Reactor Power, MWt</td>
<td>2988</td>
<td>3400</td>
</tr>
<tr>
<td>Hot Leg Temperature, °F</td>
<td>626</td>
<td>610</td>
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<tr>
<td>Number of Fuel Assemblies</td>
<td>157</td>
<td>157</td>
</tr>
<tr>
<td>Type of Fuel Assembly</td>
<td>17x17</td>
<td>17x17</td>
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<tr>
<td>Active Fuel Length, ft</td>
<td>14</td>
<td>14</td>
</tr>
<tr>
<td>Linear Heat Rating, kW/ft</td>
<td>5.02</td>
<td>5.71</td>
</tr>
<tr>
<td>R/V I.D., inches</td>
<td>157</td>
<td>157</td>
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<tr>
<td>Vessel Thermal Design Flow, gpm</td>
<td>295,500</td>
<td>299,880</td>
</tr>
<tr>
<td>Steam Generator Surface Area, ft²</td>
<td>68,000</td>
<td>125,000</td>
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<tr>
<td>Reactor Coolant Pump Flow, gpm</td>
<td>103,400</td>
<td>78,750</td>
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<tr>
<td>Pressurizer Volume, ft³</td>
<td>1400</td>
<td>2100</td>
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</table>
AP 1000:
The Passive Safety System
Passive Safety – What’s it all about?

- Passive Safety Systems utilizes naturally occurring physical phenomena such as natural circulation of air, water and steam.
- **Gravity** and gas pressure drive the flow of cooling water.
- **Natural heat transfer** occurs through conduction, convection and evaporation.

- There are no safety related pumps and motor-operated valves.
- There is no safety related ventilation system.

- Reactor safety functions are achieved without using any safety related **AC power** rely on “stored” energy.
- A few battery powered valves - 20 in total, most “fail safe” - align the passive safety systems upon actuation signals (“one time” alignment).
Passive Safety – What’s it all about? (Cont.)

- Typical PWR Safety Systems and Safety-Related Support System exist as simplified, non-safety systems housed in non-safety related structures *(defense in depth)*:
  - Diesels
  - Chilled Water
  - Component Cooling
  - Spent Fuel Cooling
  - Diesel Support Systems  
    (Fuel Oil, HVAC)  
  - Instrument Air
  - Hot Water (heating)
  - Essential Service Water
  - Residual Heat Removal
  - Startup Feedwater  
    (replaces Auxiliary Feedwater)

- The Non-Safety, Active Systems are credited in the PRA, but are *not* required for reactor safety and are *not* required to achieve NRC required CDF of $1 \times 10^{-4}$

- Severe Accident Scenarios are mitigated by In-Vessel Retention of the melted fuel. (The core is retained in a cooled reactor vessel)
Passive Decay Heat Removal

- PRHR (Passive Residual Heat Removal) normally isolated by two AOVs, fail open
  - Opening 1 AOV actuates RCS cooling via natural circulation
  - AOVs actuated by PMS and by DAS
- IRWST absorbs heat
  - Takes ~ 2 hours to heat up to saturated
  - Steaming is condensed by PCS and returned to IRWST by gutter
Passive Safety Injection
Passive Safety Injection

- Uses one time valve alignment
  - Accumulator uses check valves
  - CMT uses fail open AOV
  - ADS uses MOV for 1/2/3 and Squibs for 4
  - IRWST uses Squibs and check valves
Simplicity in Design and Safety

- No operator action for 72 hours
- AP1000 safety does not rely on AC power
  - Passive decay
    heat removal
  - Passive safety injection
  - Passive containment cooling
- Uses passive safety systems
  - Proven by extensive testing and analysis
  - Extensively reviewed by USNRC
  - No safety pumps
  - No ac power required
  - One time valve alignment - Most are fail safe
- Provides improved margins
  - Transient DNBR margin > 15%
  - No core uncovery for SBLOCA
  - Breaks up and including a Direct Vessel Injection (DVI) line (8”) break
  - No operator actions required for SG Tube Rupture
Passive Core Cooling System Operation During a Small-Break LOCA
Passive Containment Cooling System

AP1000
Ultimate
Heat Sink is
the
Atmosphere
Passive Containment Cooling Operation During a LOCA
LOCA Long-Term Cooling
Simple Severe Accident In-Vessel Retention

- AP1000 designed to retain core debris within the reactor vessel
- Provides Reliable Means of Cooling Damaged Core
  - Tests and analyses reviewed by U.S. NRC
- In PRA Core Damage Sequences
  - Cooling flow driven by natural circulation
  - Water source: In containment refueling water storage tank
  - Cooling water flow path in vessel/insulation annulus
  - Core heat transferred through RV wall
  - Water in containment removes heat from RV
  - ADS valves keep RCS pres low
  - Passive containment cooling transfers heat out of containment
  - Core debris retained inside reactor vessel
- Large release frequency: 5.9 x 10^-8 per reactor year; URD requires < 10^-6
In-vessel Retention for Severe Accident Management
Simplification of Safety Systems
Dramatically Reduces Building Volumes
All Advanced Reactors Can Achieve Low Core Damage Frequency

- Evolutionary plants achieve goals by **adding** redundant safety features
  - 4 Train Safety Injection
  - 4 Train Decay Heat Removal
  - 4 Train Containment Cooling
  - 4 Train Residual Heat Removal
  - 4 Train Diesel Generators

- Passive Plants achieve goals by **reducing** active safety features
  - No safety related pumps
  - No safety related fans
  - No safety diesels/no safety AC power
  - Small number of valves actuate passive systems
  - Natural forces provide plant safety
AP1000 Passive Safety System Design Improves Economics and Construction Schedule

- 50% Fewer Safety-grade Valves
- 35% Fewer Pumps
- 80% Less Safety-grade Pipe
- 45% Less Seismic Building Volume
- 70% Less Cable
Comparison of Seismic Category I Buildings

- Shield / Containment
- Auxiliary Building
- Fuel Area
- Diesel Generators
- Essential Service Water Pumphouse
- Emergency Fuel Oil Storage
- Refueling Water Storage Tank
AP1000 Construction Simplification

*Think: 1) more power/m³ of concrete, 2) less to decommission*

<table>
<thead>
<tr>
<th></th>
<th>Concrete, m³</th>
<th>Rebar, metric tons</th>
<th>Power, MWe</th>
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<tbody>
<tr>
<td>Sizewell B</td>
<td>520,000</td>
<td>65,000</td>
<td>1188</td>
</tr>
<tr>
<td>AP1000</td>
<td>&lt;100,000</td>
<td>&lt;12,000</td>
<td>1117</td>
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AP1000 Provides Safety and Investment Protection

U. S. NRC Requirements

Current Plants

Utility Requirements

AP1000 Results

Core Damage Frequency per Year (All Events)

- 1 x 10^{-4}
- 5 x 10^{-5}
- <1 x 10^{-5}
- 5.1 x 10^{-7}
AP 1000:
The Most Tested Reactor
AP 1000 Mature design

- AP 1000 is the **most tested reactor** (USA, Japan, Italy)
- Licensing
  - NRC *(total staff 4,000 people)* performed **independent and confirmatory plant tests** (ROSA Japan, Oregon State University)
  - NRC spent 110 man-year effort over 6 years (*W* spent 1,300 man-years…)
  - NRC asked 7,400 written questions
  - Finally NRC issued Design Certification in Dec. ‘99
- Meets UDR (US utility requirements)
- Meets EUR (European utility requirements)
Thorough Testing of AP1000 Passive Systems
- Definitely the Most Tested Reactor -

Oregon State ¼ Scale, Long Term Integral Systems Test

Full Height, Full Pressure, Integral Systems Test (SPES)

Large-Scale Containment Heat Transfer PCS Test
Full Height, Full Pressure, Integral Systems Test (SPES-2 - Italy)
APEX Facility (USA) Description

Vessel Head & ADS 4 Lines
Core Heater Installation and Baffle
PZR & SG Head

Reactor Coolant Pumps
Hot Leg, Cold Legs and Vessel
Large-Scale Heat Transfer PCS Test - USA
ULPU-2400 Config. V Full Scale Slice Testing
Confirms IVR Heat Transfer from Outside Reactor Vessel
ULPU-2400 Config. V Full Scale Slice Testing

Confirms IVR Heat Transfer from Outside Reactor Vessel
AP 1000: The Modular Construction
Modules Designed into AP1000 from the Beginning

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<tr>
<th>Module Type</th>
<th>Number</th>
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<tbody>
<tr>
<td>Structural</td>
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<tr>
<td>Piping</td>
<td>154</td>
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<tr>
<td>Mechanical Equipment</td>
<td>55</td>
</tr>
<tr>
<td>Electrical Equipment</td>
<td>11</td>
</tr>
<tr>
<td>TOTAL</td>
<td>342</td>
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</table>
Modularization – Impact on Construction

● Reduced calendar time, site labor and overall risk:
  ● Module fabrication performed parallel with civil / structural work
  ● Optimizes and levels manloading for mechanical and electrical work
  ● Site congestion reduced
● Inspections performed at fabrication shop
● Reduced on-site work
● Concrete curing and coating time is drastically reduced
Residual Heat Removal System - Pipe / Valve Module
Chemical and Volume Control System Equipment Module
Waste System Demineralizers
Integration Three Waste Modules
Structural modules

Truss Wall

Structural
CA01 Submodule Details
Schedule and Model Integration

- Intergraph design review product
- Primavera Project Planner
- Continuing interactions with software vendor
- 3D model boundaries modified to match schedule activities
- Link by activity
- Activity construction durations maintained
- Schedule improvements by logic changes only
- Visualization capability for changing activity sequences
Modular Construction Allows More To Be Done in Parallel
Result: Shorter Construction Schedule
AP 1000: Ten Plants Under Construction, China and USA
The Renaissance Is Here
New Reactor License Applications in U.S.
**The Renaissance Is Here**

**AP1000™ Projects in U.S.**

14 AP1000 units selected by U.S. utilities to date

6 AP1000 units under contract

**SCE&G**
- VC Summer 2: C.O.D. 4/1/2016
- VC Summer 3: C.O.D. 1/1/2019

**Southern Co.**
- Vogtle 4: C.O.D. 4/1/2017

**Progress Energy**
- Levy County 1: C.O.D. 7/1/2016
- Levy County 2: C.O.D. 7/1/2017
AP1000 China Projects

Two units at Haiyang
Two units at Sanmen

Construction on Schedule

<table>
<thead>
<tr>
<th></th>
<th>Fuel Load</th>
<th>COD</th>
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<tbody>
<tr>
<td>Sanmen #1</td>
<td>May 2013</td>
<td>Nov 2013</td>
</tr>
<tr>
<td>Sanmen #2</td>
<td>Mar 2014</td>
<td>Sept 2014</td>
</tr>
<tr>
<td>Haiyang #1</td>
<td>Nov 2013</td>
<td>May 2014</td>
</tr>
<tr>
<td>Haiyang #2</td>
<td>Sept 2014</td>
<td>Mar 2015</td>
</tr>
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Status of Construction Milestones

- All four NI excavations have been completed
- Sanmen 1 First Concrete Milestone completed on schedule
- Successful setting of CA20
- Auxiliary Building walls started from 66’6” to 82’6”
- Manufacturing of heavy components is on schedule
- Valve development & Demonstration program on track
- Containment Vessel final assembly is occurring

- Reactor Coolant Pump build and test plan is on schedule
- All long lead forgings have been ordered and manufacturing is underway
- Module Factory in China was operational May 31, 2008
The Renaissance Is Here
First AP1000™ Contracts in China
AP1000 - Sanmen First Concrete Pour

March 2009
Placement of the basemat concrete for Sanmen Unit 1 successfully ended March 31 after 46 hours and 58 minutes of continuously pouring 4,982 cubic meters of concrete.
The Renaissance Is Here
First AP1000™ Contracts in China

Rotation of CA-20 Module – Sanmen 5 / 2009

Size (N x E x Height): 44'-0" x 68'-9" x 68'-0"
Weight: 1,717,800 lbs.
The Renaissance Is Here
First AP1000™ Contracts in China

June 29, 2009
Project is implemented

- Westinghouse and Shaw Home Offices are staffed, initial systems and procedures are in place, and implementing Project per schedule
- Site mobilized and early phases of construction are underway

PSC certification approved March 17, 2009

Full Notice to Proceed received from Southern Nuclear Company on March 31, 2009

17 of 19 early procurements have been placed (remaining two will be placed by August 2009)
U.S. AP1000™ Projects: Project Update
Vogtle Units 3 & 4 Project Update

• Construction
  – Issued Notice of Commencement
  – Continued site mobilization, temporary trailers installed, subcontractors mobilized
  – Construction access road completed
  – Interim security control established
  – Runoff ponds complete, 100-year run-off ditch relocated
  – Preparing for plant site excavation
Vogtle 3 &4
March 23 – 30, 2009
U.S. AP1000™ Projects: Project Update
VC Summer 2 & 3 Project Update

Activities

- PSC certification of project approved February 11, 2009
- Continuing site preparations including construction access road completion, installation of catch basins, development of lay-down areas, construction city development and rail spur rough grading
- Placed purchase orders for major components including steam generators, reactor vessel, reactor coolant pumps, containment vessel, turbine generator, pressurizer, reactor coolant piping, reactor vessel internals and CRDMs
Activities (continued)

- Issued project schedule performance baseline April 1, 2009
- Initiated grading plant access road and the turn lanes at the intersection of Parr Road with State Highway 213 (main access to site)
- Received State Regulatory approval to start clearing and grading the table top in preparation for excavation (Phase 3A permit)

Upcoming Work

- Commence table top grading in May 2009
- Complete rail spur installation in June 2009
- Complete new hire and admin. support buildings in September 2009
- Complete Mayo Creek bridge construction Jan. ‘10
U.S. AP1000™ Projects: Project Update

Levy County 1 & 2 Project Update

- “Determination of Need” petition unanimously approved by Florida PSC in July 2008
- Levy County 1 & 2 COLA submitted by Progress Energy to NRC on July 30, 2008
- EPC contract signed on December 31, 2008
U.S. AP1000™ Projects: Project Update

Levy County 1 & 2 Project Update

- Consortium home office teams have been assembled and are fully engaged with Progress Energy
- Long-lead procurement of major components commenced in 2008
- Progress Energy notified consortium on April 30, 2009 of partial suspension of work
- On August 13, 2009 Progress Energy received state site certification from the Florida Power Plant Siting Board
- Expect to receive an NRC combined construction permit-operating license in late 2011 or early 2012
AP 1000
The Renaissance Has Started!

THANKS