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Applications of Nuclear Power: Seawater  
Desalination, Hydrogen Production and other  
Industrial Applications*

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*Overview of the Safety Aspects of Nuclear  
Desalination Coupling*

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**INAP**

**Bariloche, Rio Negro, Argentina**

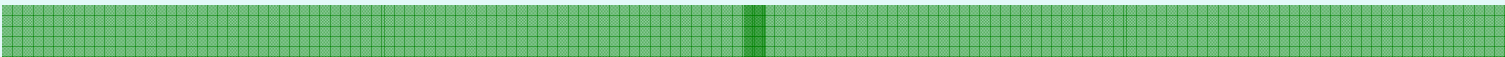
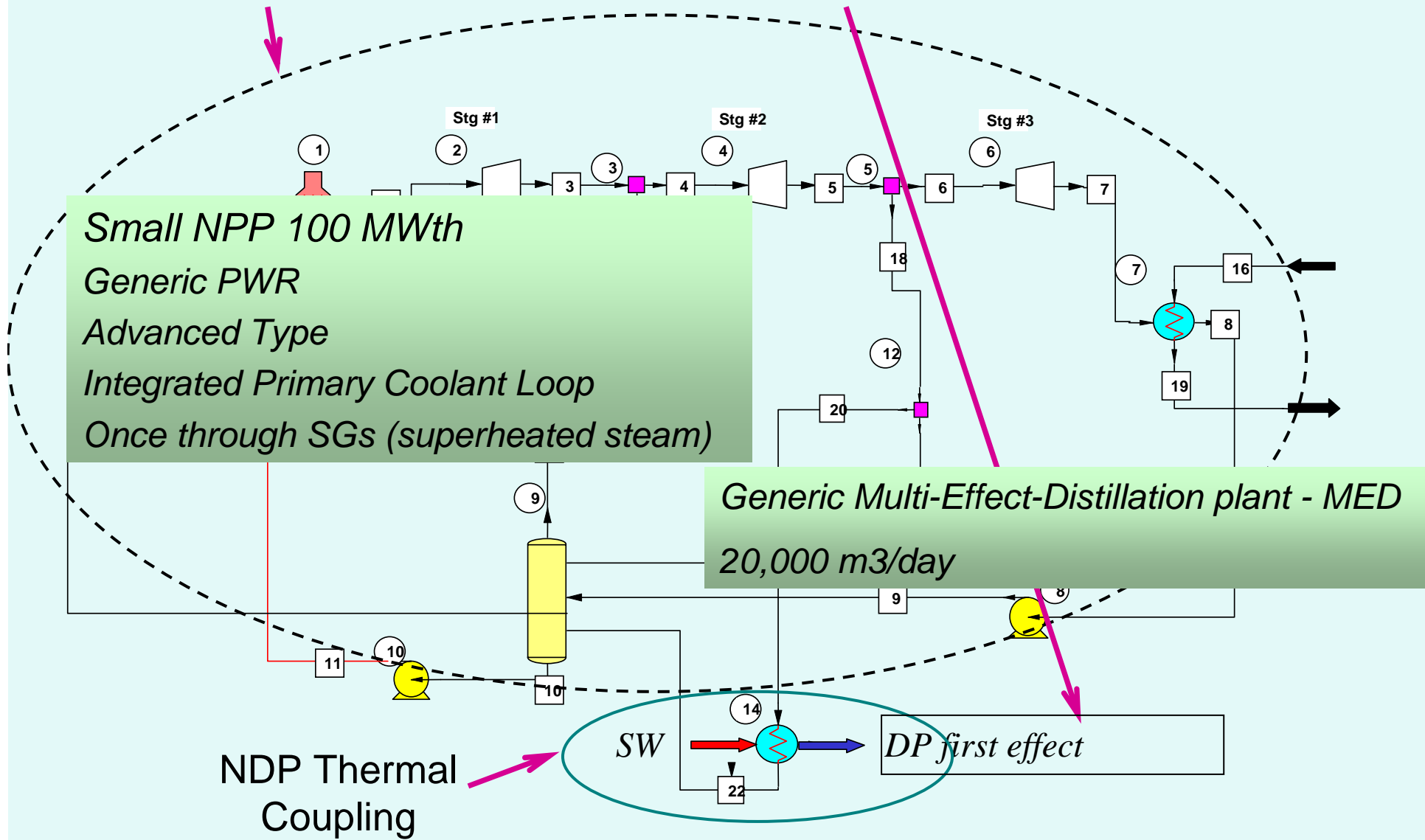
**INAP**

# *Introduction*

*Overview of the Safety Aspects of ND thermal coupling includes:*

- *Nuclear safety approach*
- *Generic issues that should be included in the NDP SAR*
- *Conceptual design of Coupling systems (two different ESFs)*

# NPP + DP = NDP



# Safety Approach

- Safety objective** of NDP coupling
- No adverse effect on the safety of the NPP ✓
  - To ensure that no hazards (different nature/higher probability) than those stated in the NPP SAR arise

The only relevant **safety function** related to a coupling system is the confinement of radioactive material (reactivity and cooling are not an issue)

The potentially radioactive source is not necessarily the reactor core but the BoP water

Safety Objective derives in **General Safety Design Requirements**

*# 1: Provision of barriers between potentially radioactive material and PW*

*# 2: Provision of features preventing radioactive material from reaching PW in case of failure sequence*

# Commonly used terms

- ✓ **PIE:** Events giving rise to failures or a sequence of failures leading to accidental conditions
- ✓ **Accidental Sequence:** the evolution of the plant starting from a PIE and according to a possible sequence of failures
- ✓ **Envelope Safety Case:** the accidental sequence with the most severe consequences
- ✓ **Critical Group:** the group of people that has the higher probability of being exposed to the effluents of the NP
- ✓ **Defence-in-Depth concept:** successive barriers providing graded (envelope) protection against transients. The levels of defence in depth are:
  - a) Prevention of deviations from normal operation/ failures;
  - b) Control of such deviations/failures to prevent AOO to reach accident conditions;
  - c) Control of the consequences of the resulting accident conditions;
  - d) Control of severe conditions including prevention and mitigation of the consequences;
  - e) Mitigation of radiological consequences

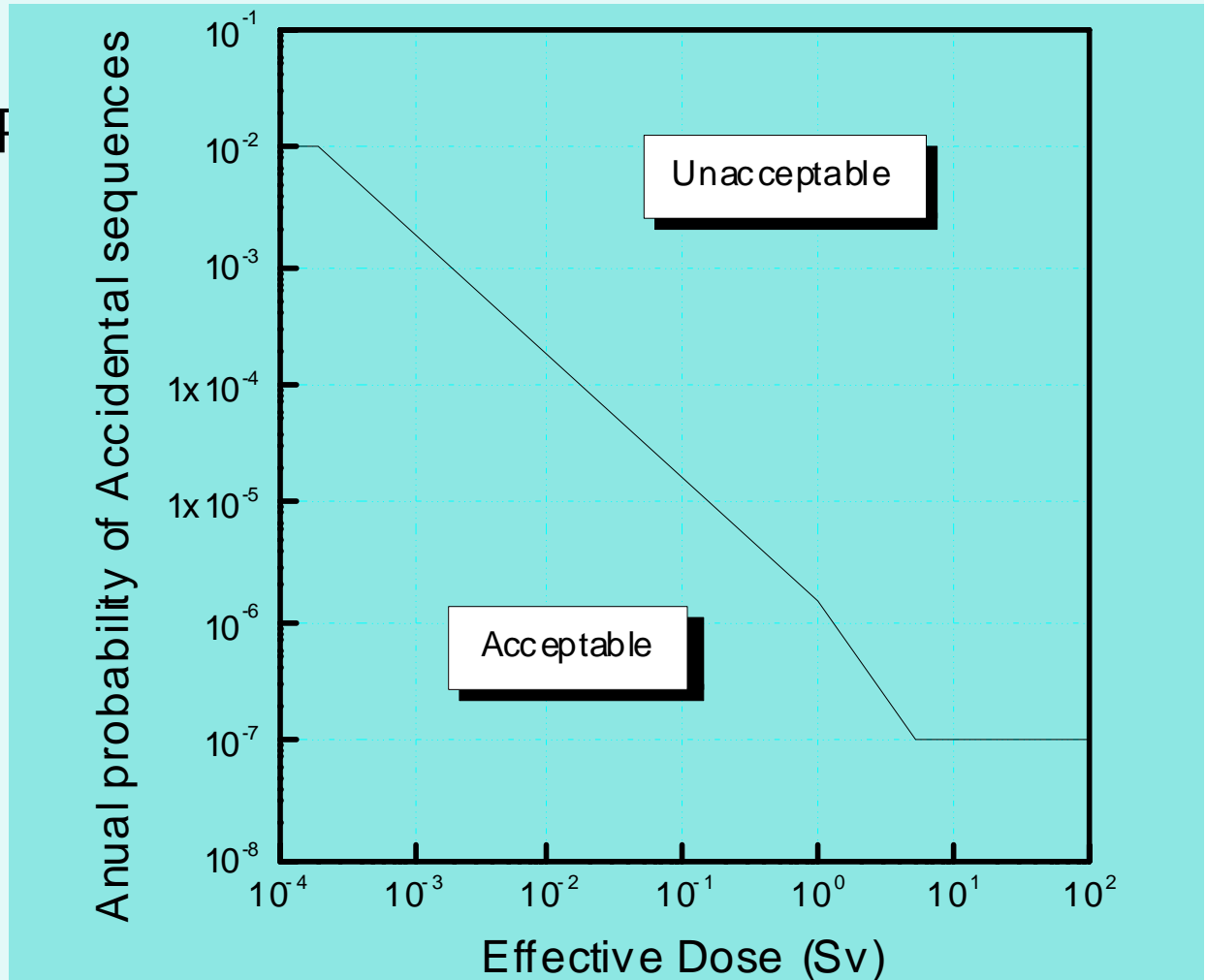
# Safety Approach - Proposed Structure of SAR

- NPP-SAR based on a deterministic safety approach
- DP coupling to a NF

Up-date style safety



Probabilistic approach



# *Safety Approach - Proposed Structure of SAR*

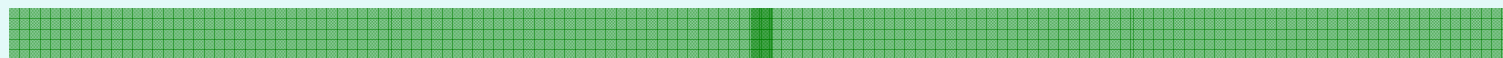
**DP** ➔ Implies a major modification to the design

- DP-SAR
- an appendix, self-standing document
  - submission does not imply a NPP-SAR revision

“NRC Standard format and content of SAR for NPP”

Adopted as a guide

**Only** to provide a uniform **format**  
for presenting the information



# *Specific Contents of the SAR for a NDP*

*Chapter 1: Introduction and General Description of the Facility*

*Chapter 2: Site characteristics*

*"Consumers group"*

*= or ≠*

*"Critical group"*

*Chapter 3: Design of Structures, Components, Equipment and Systems*

*Chapter 4: Reactor*

*Not applicable for the DP SAR*

*Chapter 5: Desalination System and Connected Systems*

*Chapter 6: Engineered Safety Features*

*Chapter 7: Instrumentation & Control*

*Chapter 8: Electric Power*

*Chapter 9: Auxiliary Systems*

*Chapter 10: Steam and Power Conversion System*

*Chapter 11: Radioactive Waste Management*    *No radioactive waste produced*



# *Specific Contents of the SAR for a NDP*

*Chapter 12: Radiation Protection*

*Changes in radioactive background of seawater*

*Intake limiting values under NPP accident conditions*

*Chapter 13: Conduct of Operations*

*Chapter 14: Initial Test Program*

*Chapter 15: Safety Analysis*

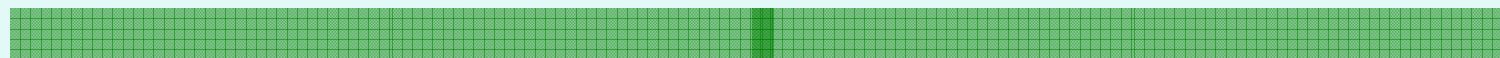
*A list of PIE presented*

*Safety cases identified and assessed*

*Chapter 16: Technical Specifications*

*Limiting Conditions for Safe Operation*

*Chapter 17: Quality Assurance*



# *Additional chapters according to SAR format index*

*The best way to provide a smooth licensing processing*

?

*Include self-standing chapters*

*Chapter (\*): Safety Objectives and Engineering Design Requirements*

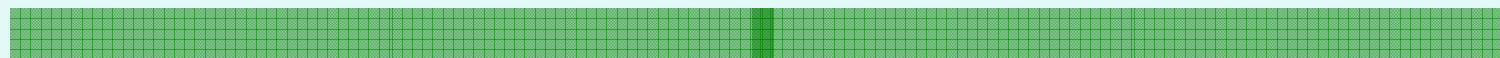
- ✓ *Description of Safety Objectives*
- ✓ *Derivation of Safety Design Requirements*
- ✓ *Development into specific Engineered Safety Features*
- ✓ *Defence-in-Depth philosophy*

*Chapter (\*): Environmental Assessment*

*If not already done, include it here*

*Chapter (\*): Decommissioning*

*Design provisions for an easy DP decommissioning task*

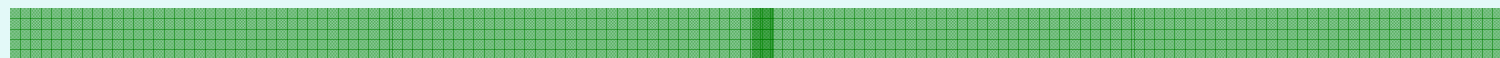


# Conceptual Safety Analysis of the Safety Case

➡ Construction of the PIE list

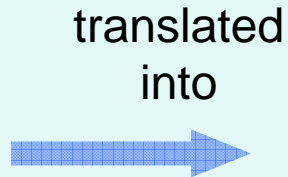
➡ Defense-in-Depth barriers

Level	Main characteristics	Safety features
	<ul style="list-style-type: none"> <li>✓ Main HX breaks</li> <li>✓ SG fails</li> <li>✓ Nuclear fuel matrix/cladding fail</li> </ul>	<ul style="list-style-type: none"> <li>water form BoP into DP</li> <li>primary coolant enters the BoP</li> <li>fission products to primary coolant</li> </ul>
<i>Highly unlikely</i>	<i>NPP detection means</i>	<i>Severe contamination</i>
	<ul style="list-style-type: none"> <li>✓ Main HX leaks</li> </ul>	<ul style="list-style-type: none"> <li>water form BoP into DP</li> </ul>
<i>Quite likely</i>		<i>Weak contamination within the DP</i>

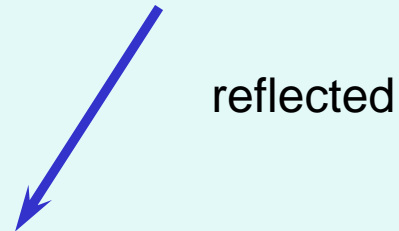


# Summary

**Safety objective**  
of NDP coupling



**GSDR**



Conceptual design of a Coupling System = Confinement of radioactive material

Following defence-in-depth principles



*ESF*

Potable Water Monitoring

Pressure Reversal in HX

# INVAP contributions and findings on PWM

**Design key issue:** the measuring time needed to reliably detect radioactive levels below acceptable limit

Measuring  $\gamma$  in arrangements of Marinelli devices

Expertise handling statistics (sensitivity, precision, confidence, design margins), admissible spurious trip frequency, I&C and process design

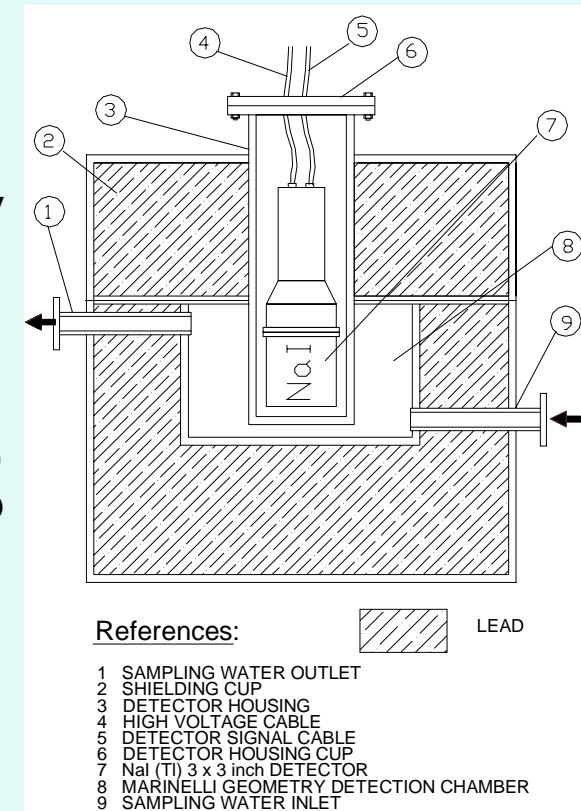
The allowable activity for PW (e.g. WHO) and data on the available sensors one gets the minimal Measuring Time for the sampling and then the hold up requirement (tank volume)

**Number of tanks:  $N = \text{or} > 3$**

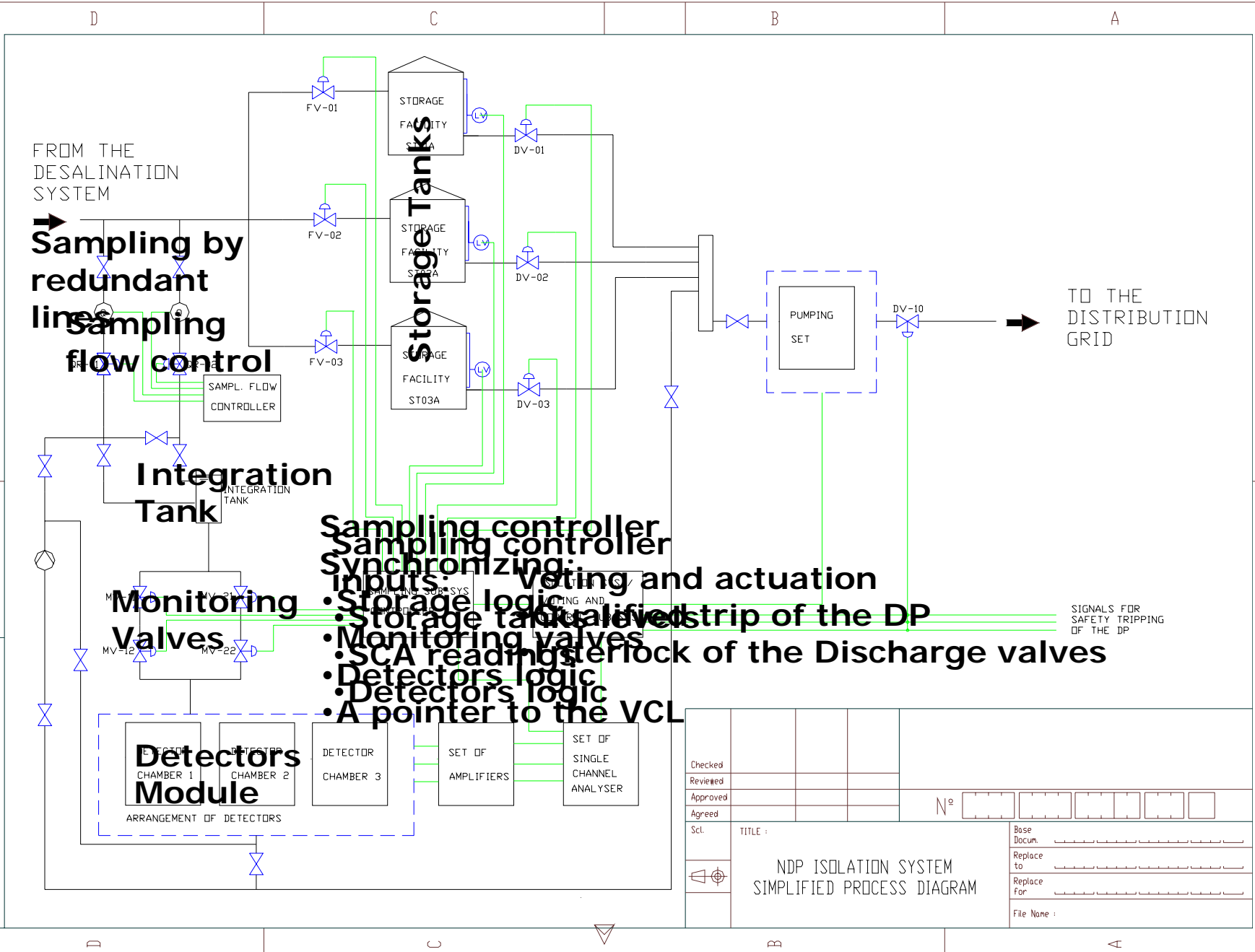
**Hold-up time required ~ 60 minutes**

For a typical MED DP of 20.000 m<sup>3</sup>/day, preliminary estimations of minimum scope Isolation System is ~ 800.000 U\$S and the impact would be bigger for smaller plants

Typical MED unit cost (900 U\$S/m<sup>3</sup>/day), 18 MU\$S for the assumed DP



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**Inputs:**

- Sampling controller
- Sampling controller synchronizing
- Storage tank level
- Monitoring valves
- SCA readings
- Detectors logic
- Detectors logic
- A pointer to the VCL

**Outputs:**

- Voting and actuation
- Strip of the DP
- Interlock of the Discharge valves

Checked									
Reviewed									
Approved									
Agreed									
Set	TITLE :						N°		
	NDP ISOLATION SYSTEM SIMPLIFIED PROCESS DIAGRAM						Base Docum. _____		
							Replace to _____		
							Replace for _____		
							File Name : _____		





## *contributions and findings on PR*

***Design key issue:*** BoP side pressure < DP side pressure

The coupling is performed by a pressurised clean-water Intermediate Loop (IL), joining the Main HX (with BoP) and the evaporator (with DP)

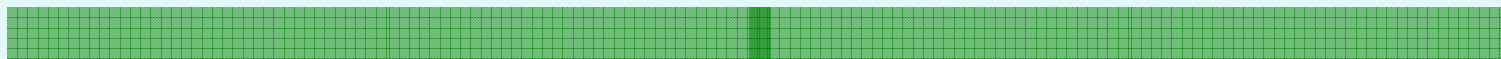
Pressure sensors monitor the  $\Delta P$  between the sides of the Main HX (barrier)

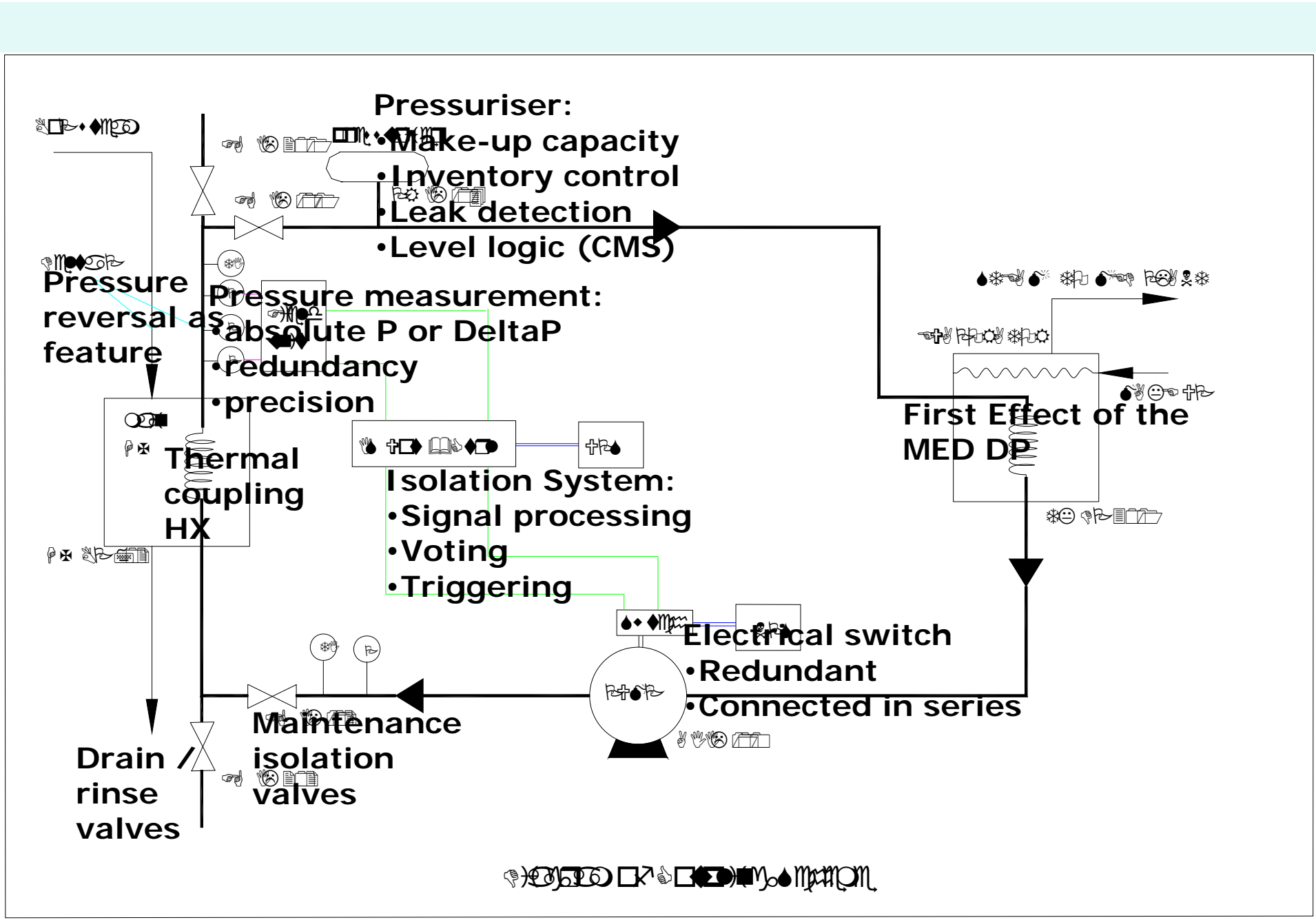
Expertise handling statistics (sensitivity, precision, confidence, design margins), admissible spurious trip frequency, I&C and process design

Technology requirements for the IS are low (only of-the-shelf equipment)

For the assumed MED DP the estimation of 34.000,- U\$S for the complete IS is negligible against the DP cost

*The impact on the NDP Project by adding an IL itself still remains as a  
an open issue*

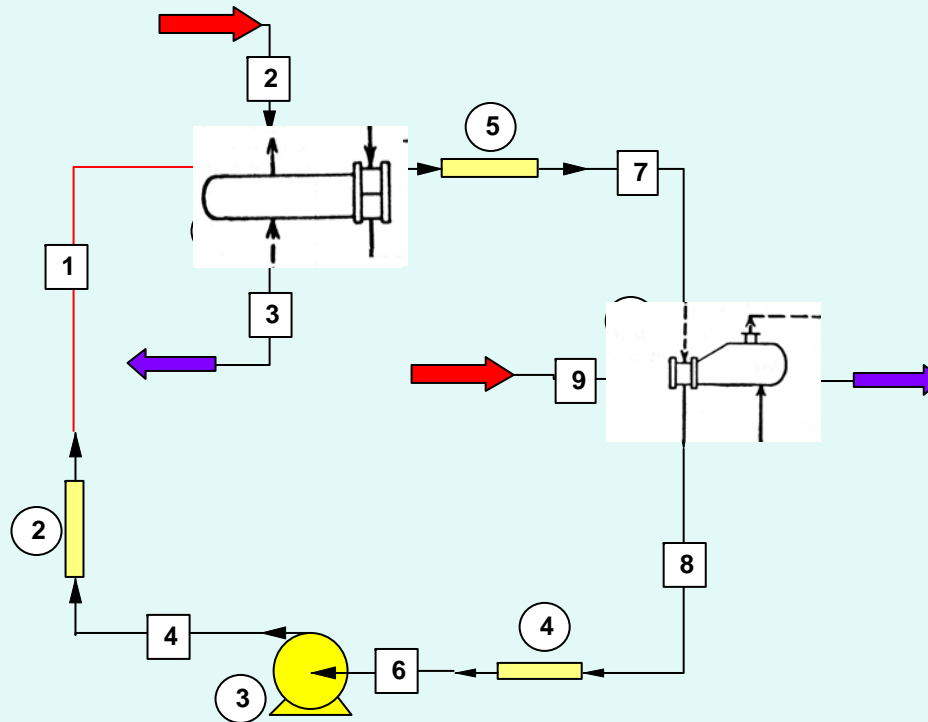






# Intermediate Loop w/ Pressure Reversal

Modelling w/Chemcad



Stream #	Turb-extr-3	Steam Cond.	I-Loop-Cold-HP	I-Loop-Hot	I-Loop-Cold-LP	MED-Feed	MED-Steam
T [C]	158.9	<b>134.5</b>	134.4	151.1	134.4	130.0	<b>132.2</b>
P [bar]	6	5.96	<b>9.00</b>	8.67	8.28	2.75	<b>2.696</b>
X [-]	1	0	0	0	0	0	<b>1</b>
F [kg/h]	11500	11500	<b>350000</b>	350000	350000	11610	<b>11610</b>

# Findings

## Reference case

Stream #	Turb-extr-3	Steam Cond.	I-Loop-Cold-HP	I-Loop-Hot	I-Loop-Cold-LP	MED-Feed	MED-Steam
T [C]	158.9	<b>134.5</b>	134.4	151.1	134.4	130.0	<b>132.2</b>
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X [-]	1	0	0	0	0	0	<b>1</b>
F [kg/h]	11500	11500	<b>350000</b>	350000	350000	11610	<b>11610</b>

The use of a High

*Turbine extraction at 4 bar instead of 6 bar*

thermal sol

*Reduces efficiency*

thermal coupling

Stream #	Turb-extr-3	Steam Cond.	I-Loop-Cold-HP	I-Loop-Hot	I-Loop-Cold-LP	MED-Feed	MED-Steam
T [C]	143	133.7	138.0	141.5	138.0	130	<b>129.8</b>
P [bar]	4	3.99	6.98	6.58	6.36	2.75	<b>2.68</b>
X [-]	1	0	0	0	0	0	<b>1</b>
F [kg/h]	11500	11500	1500000	1500000	1500000	10400	<b>10400</b>

# Findings

## Reference case

Stream #	Turb-extr-3	Steam Cond.	I-Loop-Cold-HP	I-Loop-Hot	I-Loop-Cold-LP	MED-Feed	MED-Steam
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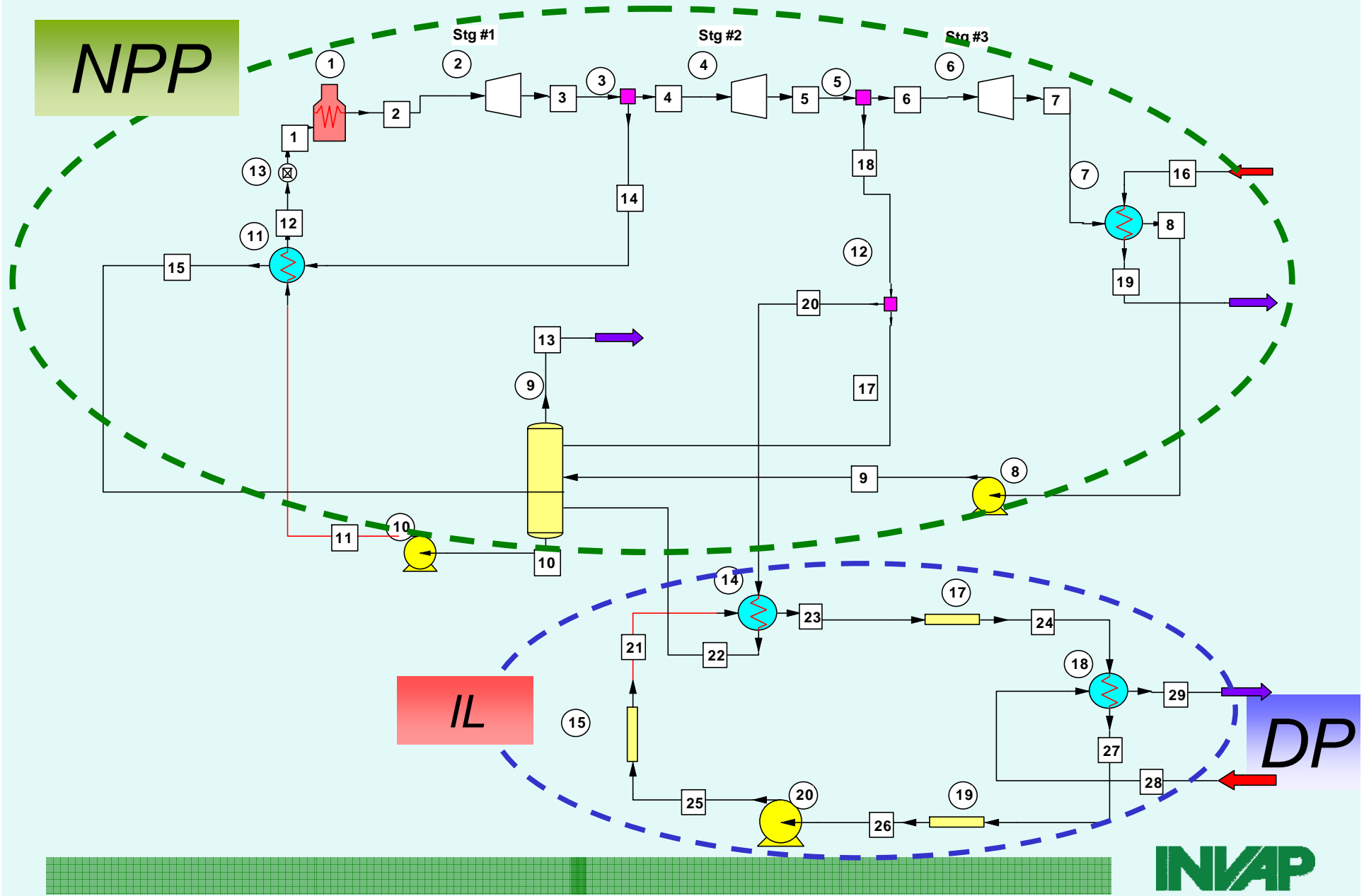
If the increase in distance is of magnitude

*Higher pressure > Pumping power*

*Same efficiency*

Stream #	Turb-extr-3	Steam Cond.	I-Loop-Cold-HP	I-Loop-Hot	I-Loop-Cold-LP	MED-Feed	MED-Steam
T [C]	158.9	134.5	134.5	151.2	134.4	130.0	<b>133.6</b>
P [bar]	6	5.96	<b>10.35</b>	9.0	7.57	2.75	<b>2.69</b>
X [-]	1	0	0	0	0	0	<b>1</b>
F [kg/h]	11500	11500	350000	350000	350000	11610	<b>11610</b>

# Idea of the Model



# *Conclusions*

- ✓ Commonly accepted techniques used for deterministic safety analysis of NPP can be applied to NDP
- ✓ The SAR of DP can be presented as a self-standing appendix acting as a complement of the NPP SAR
- ✓ The development of ESF for the thermal coupling of NDP is an issue technically solvable within Safety Guidelines
- ✓ There is not a universal IS, the optimal solution requires safety expertise
- ✓ The impact of the design solution of IS and ESF (in terms of efficiency, for example) should be considered during the first stages of a NDP when drafting user requirements



*attention*



**INVAP**