

Momentum Transport in Electron-Dominated Spherical Torus Plasmas

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Abstract. The National Spherical Torus Experiment (NSTX) operates between 0.35 and 0.55 T, which, when coupled to up to 7 MW of neutral beam injection, leads to central rotation velocities in excess of 300 km/s and ExB shearing rates up to 1 MHz. This level of ExB shear can be up to a factor of five greater than typical linear growth rates of long-wavelength ion (e.g., ITG) modes, at least partially suppressing these instabilities. Evidence for this turbulence suppression is that the inferred diffusive ion thermal flux in NSTX H-modes is often at the neoclassical level, and thus these plasmas operate in an electron-dominated transport regime. Analysis of experiments using $n=3$ magnetic fields to change plasma rotation indicate that local rotation shear influences local transport coefficients, most notably the ion thermal diffusivity, in a manner consistent with suppression of the low- k turbulence by this rotation shear. The value of the momentum diffusivity as inferred from steady-state momentum balance is found to be larger than the neoclassical value. Analysis of perturbative experiments indicate inward pinch velocities up to 40 m/s and perturbative momentum diffusivities larger by a factor of several than those values inferred from steady-state analysis. The inferred pinch velocity values are consistent with values based on theories in which low- k turbulence drives the inward momentum pinch. Thus, in STs, while the neoclassical ion energy transport effects can be relatively high and dominate the ion energy transport, the neoclassical momentum transport effects are near zero. This means that any residual low- k turbulence dominates the momentum transport.

1. Introduction

Plasma rotation in magnetic confinement devices plays a critical role for optimizing fusion plasma performance through shear stabilization of both the microturbulence that is associated with heat and energy transport as well as macroscopic magnetohydrodynamic (MHD) instabilities. In particular, ExB shear is believed to play a role in stabilizing both low- k turbulence from Ion Temperature Gradient (ITG) and Trapped Electron Modes (TEM) [1-3], and has recently been associated with a suppression of high- k turbulence from Electron Temperature Gradient (ETG) modes [4]. Additionally, ExB shearing has been noted to suppress both internal MHD modes [5] as well as external, Resistive Wall Modes [6, 7]. It has been only relatively recently that studies of the sources of plasma rotation and momentum transport have been studied, and, therefore, the understanding of the underlying physics of these is still developing. Understanding the source of the momentum transport, and how it scales to larger devices operating at lower collisionality, is critical to the performance of future ST-based Fusion Energy Development devices such as an ST-based Component Test Facility, as well as to conventional aspect ratio devices such as ITER, which is also expected to operate in electron-dominated transport regimes.

The Spherical Torus (ST), or low aspect ratio tokamak, characteristically operates at sub-Tesla toroidal fields, leading to high ExB rotation shearing rates. In particular, the National Spherical Torus Experiment (NSTX) operates between 0.35 and 0.55 T, which, when coupled to up to 7 MW of neutral beam injection, leads to central rotation velocities in excess of 300 km/s and ExB shearing rates up to 1 MHz. This level of ExB shear can be up to a factor of five greater than typical linear growth rates of long-wavelength ion (e.g., ITG) modes, suppressing these instabilities partially if not completely, often leading to the development of

ion Internal Transport Barriers in low density L-modes. Further circumstantial evidence for this turbulence reduction is that the inferred diffusive ion thermal flux in NSTX H-modes is typically at the neoclassical level, and thus these plasmas operate in an electron-dominated transport regime. In fact, possibly because of the believed sub-dominance of the low-k turbulence, the energy confinement scalings in NSTX were found to differ from those at higher aspect ratio. In NSTX H-modes, the energy confinement time, τ_E , scales as $\sim B_T^{0.9} I_p^{0.4}$, with the strong B_T scaling controlled primarily by the variation of the electron transport in the gradient region ($r/a > 0.4$), while the weak I_p scaling is dominated by the variation in the ion transport, which is essentially neoclassical in that region [2, 3], as determined by NCLASS [8] and the non-local GTC-NEO [9] codes.

This paper will cover two main topics: the effect of rotation and rotation shear on plasma energy confinement and local transport, and the study of momentum transport properties in both steady-state and by using perturbative techniques, augmenting previous work on NSTX on this topic [10]. It is found that there is a direct connection between the local rotation shear and the inferred local ion thermal diffusivity, with the ion thermal diffusivity changing in a fashion consistent with reduction of low-k turbulence by the rotation shear. Further the steady-state momentum transport properties in the NSTX electron-dominated regime can be different than those at higher aspect ratio, with $\chi_i \gg \chi_\phi$, although the ion energy and momentum diffusivities generally scale with one another in the gradient region. Perturbative experiments using neutral beam and applied $n=3$ magnetic field pulses near the edge to change the plasma rotation in the core and outer region respectively, indicate the existence of inward momentum pinches that are consistent with predications from theories based on a low-k turbulence drive. While the neoclassical ion heat transport is larger than that driven by low-k turbulence for sufficient ExB shear, the neoclassical momentum diffusivity is near zero. This means that the principal source of momentum transport could be due to residual low-k turbulence.

2. Effect of Rotation and Rotation Shear on Plasma Performance

NSTX is equipped with the means of applying low-n ($n=1$ to 3) magnetic fields to the plasma edge using a set of six coils situated on the midplane outside the vacuum vessel [6]. These applied fields can be used statically or dynamically to correct low-n error fields, and/or to feedback suppress low-n MHD mode activity including saturated or growing RWM modes. This set of coils has been used in an $n=3$ configuration to apply non-resonant magnetic perturbations (NRMP) to modify the plasma rotation. The maximal torque from these fields, as inferred by local changes in plasma rotation, is observed to be near $R=1.3$ m, which is in agreement with calculations of the torque due to Neoclassical Toroidal Viscosity in the presence of these applied perturbed fields [11]. By changing the rotation and rotation profile (gradient) locally, it is possible to study their effect and relation to both the global energy confinement and local heat transport properties of the plasma.

The time evolution of a representative series of 4 MW neutral beam heated H-mode discharges for which the $n=3$ fields were applied at various strengths is shown in Fig. 1. These plasmas had $I_p=0.9$ MA and $B_T=0.45$ T, and they were run in the Lower Single Null configuration with an elongation of 2.4. The top panel of the figure shows the level of current in the NRMP coils for the $n=3$ configuration. 500 A/turn leads to a ΔB_r of several Gauss in the plasma. The $n=3$ applied fields were changed at $t=500$ ms; prior to that time, the current in the coils is optimized to correct the $n=3$ intrinsic error field and to feedback on low-frequency $n=1$ MHD modes. The plasma rotation in NSTX is measured by the charge exchange recombination spectroscopy (CHERS) diagnostic, which is based on measurement of the carbon impurity. NCLASS

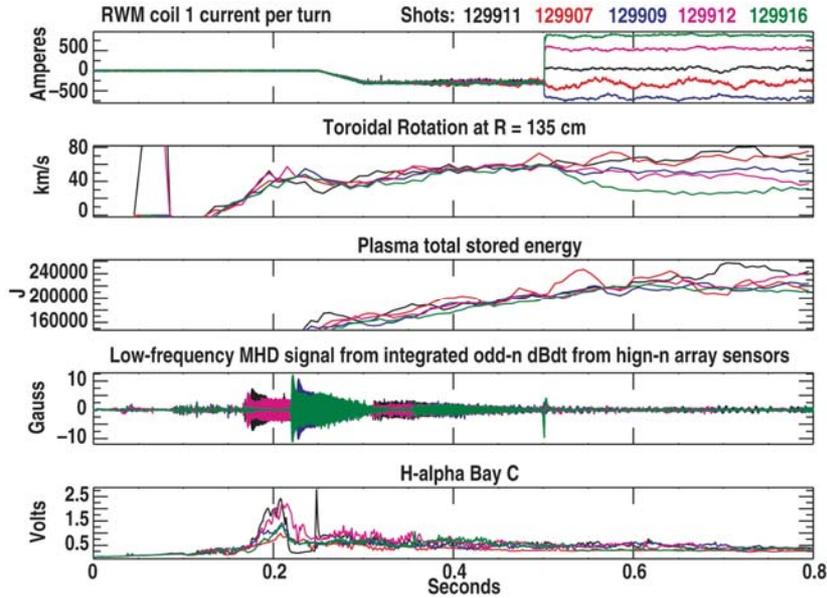


Fig. 1 Time traces for 5 H-mode discharges with application of different levels of $n=3$ braking fields. Shown from the top is the $n=3$ coil current, toroidal rotation at $R=1.35$ m, plasma stored energy calculated by EFIT, low-frequency MHD activity, and the D_α traces.

calculations indicate only a minor difference ($<5\%$) between the carbon rotation and the computed main ion (D^+) rotation. The rotation at $R=1.35$ m is seen to change with the application of the $n=3$ fields (second panel from top), with each discharge reaching a new rotational equilibrium in this MHD- and ELM-free period (bottom two panels). The rotation ranges from 25 km/s to close to 80 km/s over the range of applied $n=3$ fields. The plasma stored energy (middle panel, note the suppressed zero in the ordinate) shows little variation with the different levels of plasma rotation.

In Fig. 2 is shown the stored energy of the plasma plotted as a function of plasma angular velocity, Ω (Fig. 2a) and angular velocity shear, $\nabla\Omega$ (Fig. 2b) for the discharges shown in Fig. 1 as well as other discharges in this experimental scan. The figure shows the electron (red), ion (blue) and total stored energy (including fast ions) for the discharges in the scan, when the discharge have achieved rotation equilibrium at $t=0.72$ s. There appears to be only a slight, if any increase of stored energy in the thermal plasma species with either increasing rotation or rotation shear, although slightly more of an effect is seen in the total stored energy, where there is a comparable increase in fast ion to total plasma stored energy.

The change in rotation and rotation shear is maximum, however, in just a limited spatial range of the plasma, in these cases in the region of $r/a \sim 0.6$ to 0.9 . Consequently, it is not readily apparent that a change in these parameters would affect the plasma on a global scale. Fig. 3 shows the ion (left panel) and electron (right panel) thermal diffusivities for three representative discharges that span the range from minimum (green) to maximum (red)

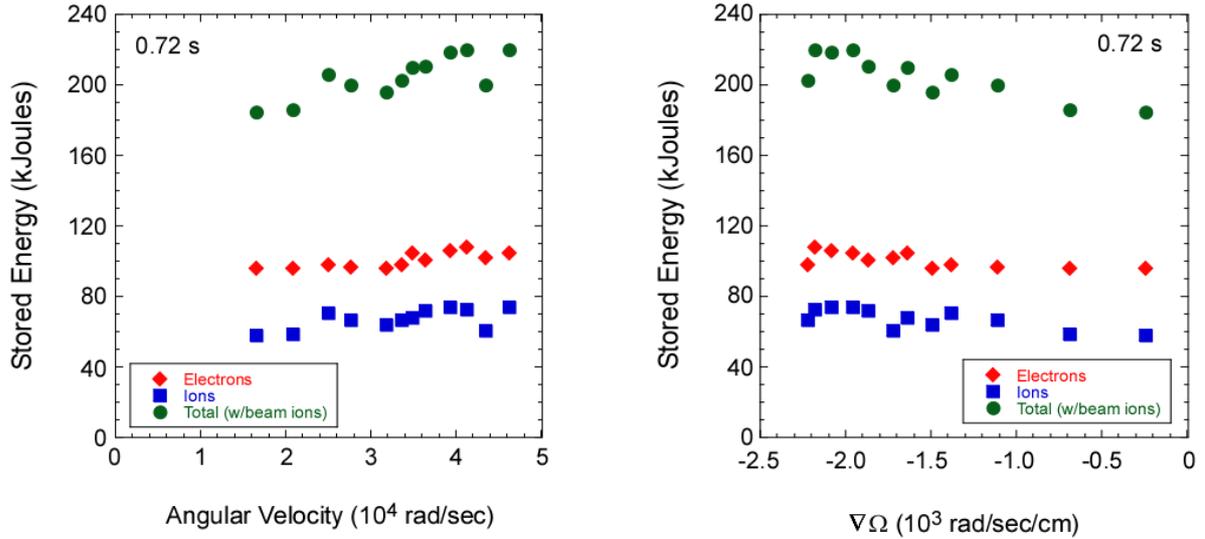


Fig. 2 Stored energy of ions, electrons, and total (including fast ions) as a function of Ω (left panel) and $\nabla\Omega$ (right panel).

rotation (2 to 6×10^5 rad/s) and rotation shear (0.2 to 2.4×10^3 rad/s/cm) in the region near $r/s=0.75$, obtained by maximum and no applied braking respectively. The thermal diffusivity values in the region near $r/a=0.75$ to 0.85 show a clear reduction with increasing Ω and $\nabla\Omega$. This is particularly true in the ion channel, where there is a factor of almost three difference between the maximum and minimum Ω and $\nabla\Omega$ cases. For the ions, the ratio of

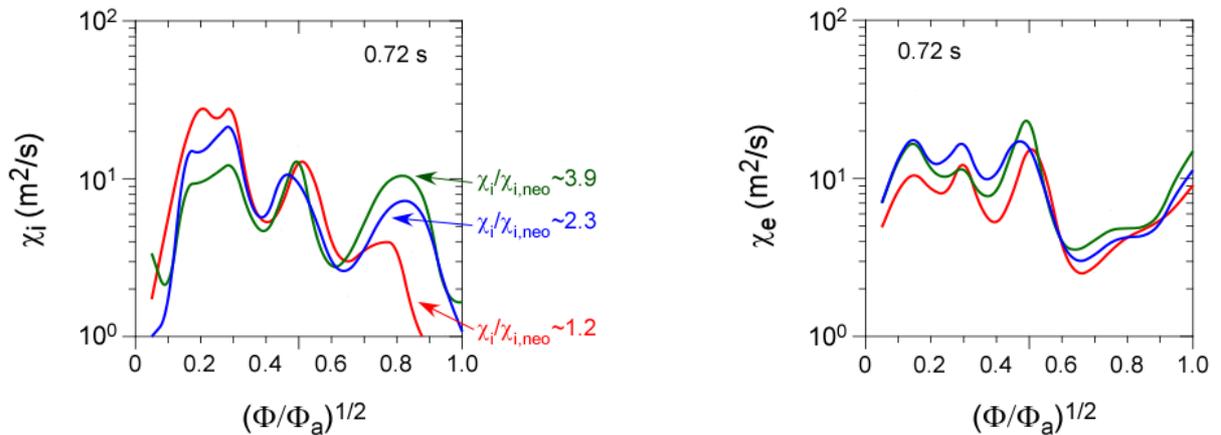


Fig. 3 Ion (left) and electron (right) thermal diffusivities over a range of Ω and $\nabla\Omega$. The green and red curves are from the discharges with minimum and maximum Ω and $\nabla\Omega$ respectively.

$\chi_i/\chi_{i,neo}$ increases from 1.2 to 3.9 with decreasing Ω and $\nabla\Omega$. Statistically, this can be seen in Fig. 4, where the $\chi_{i,e}$ values at $r/a=0.75$ are seen to decrease with increasing Ω and $\nabla\Omega$.

This dependence can be understood in terms of the degree of suppression of low-k turbulence by rotation (ExB) shear. Fig. 5 shows results from linear GS2 [11] gyrokinetic calculations of the growth rates of low-k turbulence in the minimum and maximum Ω and $\nabla\Omega$ cases. While the linear growth rates for the two cases are similar (solid curves), the ExB shearing rates are not, with the maximum Ω and $\nabla\Omega$ case having ω_{ExB} a factor of five higher than the linear growth rate (red), and the minimum case (green) having the two rates comparable. This result

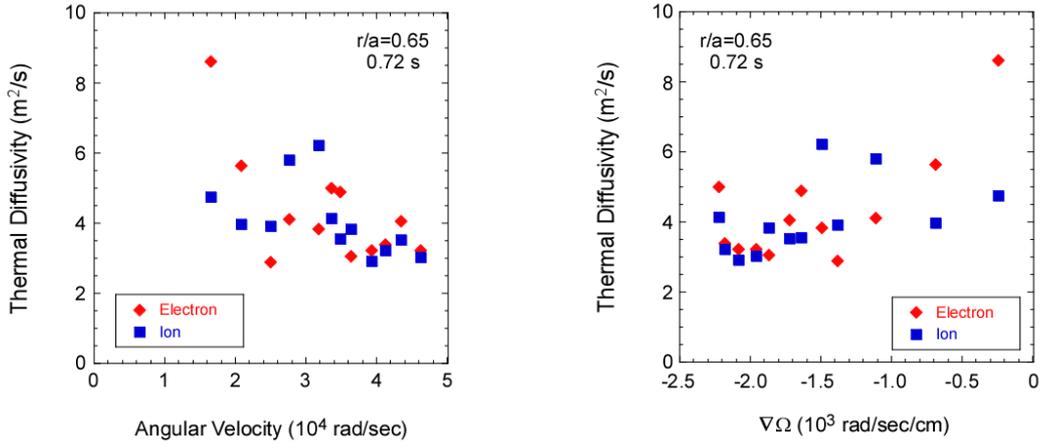


Fig. 4 Statistical summary of the dependence of χ_i and χ_e on Ω and $\nabla\Omega$.

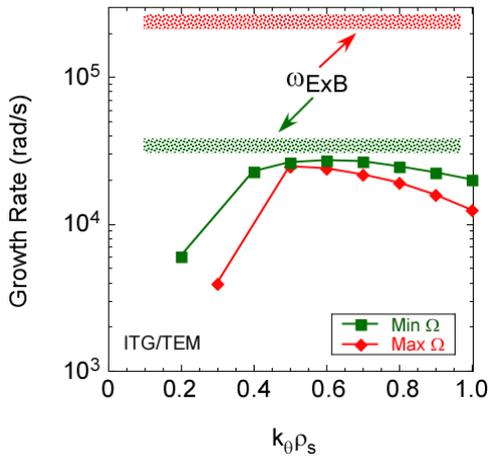


Fig. 5 Linear growth rates from GS2 along with ExB shearing rates for the two extreme discharges.

Values of χ_i and χ_ϕ from TRANSP steady-state analysis for $r/a \sim 0.65$ in both L- and H-modes are plotted versus one another in Fig. 6. The neutral beams are assumed to be the only source of torque in the steady-state analysis, which also assumes implicitly that the momentum pinch is zero. The figure shows that while there does appear to be a statistical coupling between χ_i and χ_ϕ at this radius, $\chi_i > \chi_\phi$ by as much as over an order of magnitude. There is no coupling between the two diffusivities farther into the central region of the plasma, and there is no statistical coupling between χ_e and χ_ϕ at any radius, with $\chi_e \gg \chi_\phi$.

suggests more of a reduction in the low-k (ITG/TEM) turbulence in one case than the other, and it is consistent with the increase $\chi_i/\chi_{i,neo}$ from ~ 1 to ~ 4 going from the case with $\omega_{ExB} \gg \gamma_{lin}$ to the one with $\omega_{ExB} \sim \gamma_{lin}$.

2. Momentum Transport Studies

The results in Section 1 showed that the ExB shear can be used to control the ion transport, and that with sufficient shear the ion transport is near neoclassical and the transport loss is dominated by the anomalous electrons. In this section the source of momentum transport is studied using both steady-state and perturbative techniques.

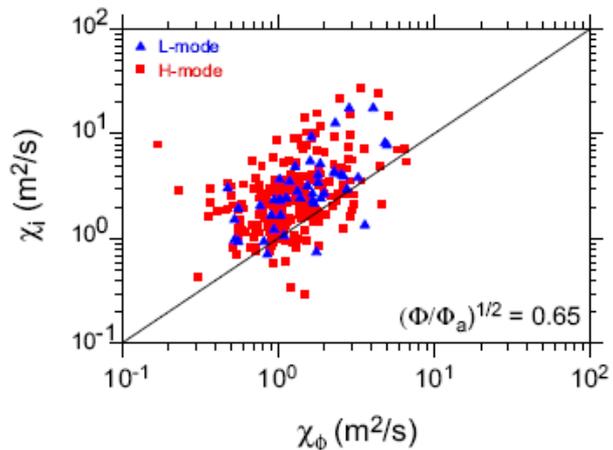


Fig. 6 χ_i vs χ_ϕ as determined from steady-state analysis

Because of the coupling between χ_i and χ_ϕ in this region of the plasma, where χ_i can often be close to neoclassical, it is natural to explore whether χ_ϕ is also controlled by neoclassical processes. GTC-NEO has been used to address this, with the results for H-modes shown in Fig. 7. Plotted in this figure are the

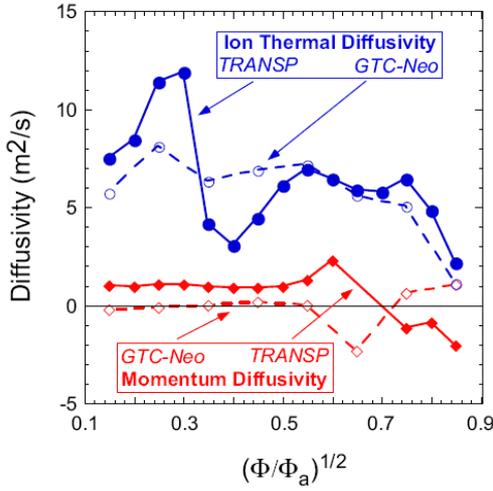


Fig. 7 Experimentally inferred values of χ_i and χ_ϕ compared to the neoclassical values as computed by GTC-NEO.

experimentally inferred χ_i and χ_ϕ (solid blue and red lines respectively) along with the values determined from GTC-NEO (dashed blue and red lines). For this H-mode discharge, it can be seen that while the experimentally inferred χ_i is essentially at the neoclassical level across the plasma, $\chi_\phi \gg \chi_{\phi,neo}$, which is essentially zero. This result is robust, even when $\chi_i/\chi_{i,neo} > 1$, as in high density L-mode plasmas. This result indicates, therefore, that the primary driver for the momentum transport must be something other than neoclassical processes. It is worthwhile to note that $\chi_{i,neo}$ is relatively large, and at least as large as χ_ϕ .

To develop a fuller picture of momentum transport, both the momentum diffusivity and any possible momentum pinch must be determined. To accomplish this, torque perturbations were applied using either 200 ms neutral beam pulses of approximately 2 MW (in addition to steady 2 MW injection), which changed rotation primarily in the core, or 50 ms pulses of n=3 braking fields, which changed the rotation primarily in the outer (gradient) region. In the first scenario, the rotation in the core responded immediately, similar to the response shown in Fig. 1, while in the latter scenario the rotation near R=1.3 m responded immediately. Using the measured angular velocity profiles and a determination of the neutral beam torque, the angular momentum balance was calculated in TRANSP. For the NBI pulses cases, the chosen time of analysis began with the NBI pulse, while for the application of the n=3 braking fields, the time of analysis began immediately after the braking pulse was turned off. This period was chosen in the latter scenario since the NB induced torque, the only known torque at that time, is more readily calculated than that from the applied n=3 fields [10].

The momentum flux, Γ_ϕ , including both a diffusion and a pinch term, can be written

$$\Gamma_\phi = -mnR\chi_\phi \frac{\partial V_\phi}{\partial r} + mnRV_{pinch} V_\phi,$$

where v_ϕ is the measured toroidal velocity and Γ_ϕ is calculated in TRANSP. χ_ϕ and v_{pinch} are modelled by assuming they are constant in time during the analysis period, and solving the above equation using a nonlinear least squares fit [for details see Ref. 10]. In order to determine both χ_ϕ and v_{pinch} independently, the change in v_ϕ and ∇v_ϕ in response to the change in torque have to be decoupled. This requirement was satisfied in the outer region of the plasma for the of the n=3 field pulses, but only for a limited spatial region in the core for the NBI pulses.

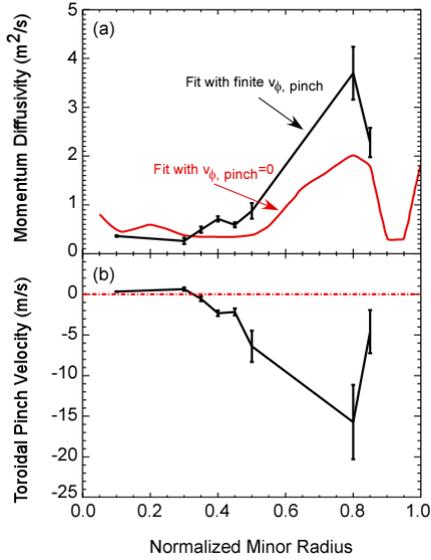


Fig. 8 (a) momentum diffusivity and (b) pinch velocity inferred using $n=3$ magnetic braking. The inferred diffusivity assuming $v_{pinch}=0$ is shown in red.

in the plasma, Peeters [13] and Hahm [14]. Both theories find that $v_{pinch} \sim \chi_{\phi}/R$, although Peeters claims an additional dependence on the density gradient scale length, L_n . A comparison of v_{pinch} as computed by these theories to the experimentally inferred values for the outer region of the plasma is shown in Fig. 9. The red points in the figure show the v_{pinch} computed by the Hahm theory, while the blue points show those computed with the Peeters theory, both sets as functions of the experimentally inferred pinch velocities. The best fits through the data (forcing the fit through zero) is shown by the color-coded lines and fit equations in the figure. In general, both theories indicate reasonable agreement with the inferred values, although Peeters theory, while introducing more scatter, appears to fit better especially for larger v_{pinch} . This is seen by the fact that the best fit through the Peeters points have a slope of ~ 1.1 , as compared to 0.64 for the Hahm points. The better fit by the Peeters theory is due to the presence of the L_n term. The larger v_{pinch} (>20 m/s) typically occur for lower L_n (~ 0.1 to 0.2 m), and it is in this range where Peeters does a better job fitting the experimental points. Both theories do equally well for lower v_{pinch} , where $L_n=0.2$ to 1 m. In the inner region, the comparison between the inferred pinch velocities and those calculated by either theory was poor, for all L_n . This result is also consistent with linear gyrokinetic calculations indicating that ITG/TEM modes are unstable in the outer region, but stable in the core in these plasmas.

4. Summary and Discussion

The results for one such case, in the outer portion of the plasma, is shown in Fig. 8. The momentum diffusivity is shown for a case with finite v_{pinch} , and for the case where v_{pinch} is assumed to be zero in solving the above equation. It can be seen that the χ_{ϕ} with non-zero v_{pinch} can be several times larger than that when v_{pinch} is assumed to be zero. This would mean that the large ratio of χ_i/χ_{ϕ} shown in Fig. 6 would trend lower when a pinch term is included in the analysis. Furthermore, the inward pinch can be significant (bottom panel), in this case with a value of up to 20 m/s in the region from $r/a=0.6$ to 0.8 . Other cases show v_{pinch} up to 40 m/s. I_p and B_T scans revealed a decrease in χ_{ϕ} with B_T in the outer region of the plasma, but little dependence of χ_{ϕ} or v_{pinch} with plasma current. No dependences were observed in the core region.

There have been two theories suggesting that the source of the momentum pinch is low-k turbulence

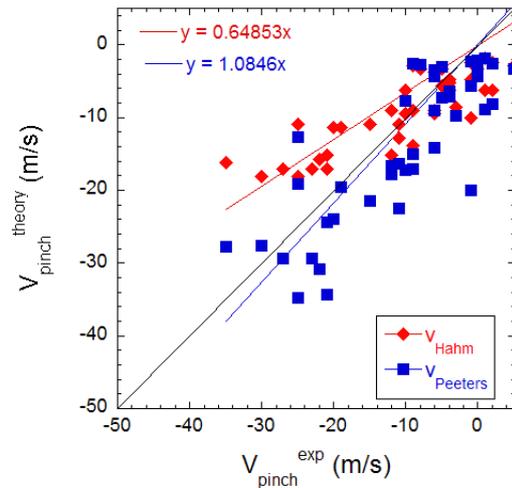


Fig. 9 V_{pinch} as computed by the Hahm (red) and Peeters (blue) theories vs experimentally inferred values.

The application of $n=3$ non-resonant magnetic perturbations to modify the plasma rotation was shown to change not only the rotation locally in the region of maximum torque ($R \sim 1.3$ m), but also to change the rotational shear. Because the torque perturbation is limited spatially, the effect on the overall stored energy of the plasma is small. However, the modification in rotation shear is seen to affect the local χ_i ; there is an obvious increase in χ_i and $\chi_i/\chi_{i,neo}$, the latter to values >1 , with decreasing rotational shear. This trend is consistent with decreasing suppression of low- k turbulence by ExB shear. As the shear decreases to values near the linear growth rate of the ITG/TEM modes, the ion thermal diffusivity becomes more anomalous. For $\omega_{ExB} \gg \gamma_{lin}$, the ion transport is near neoclassical, due to at least partial suppression of these low- k modes.

The steady-state momentum diffusivity can be over an order of magnitude greater than χ_i , but the ratio χ_i/χ_ϕ trends closer to 1 (but still remains a factor of several), when an inward pinch, which can be significant, is taken into account. The magnitude of the inward pinch in the outer region is consistent with values determined from theories based on low- k turbulence, while the agreement is poor in the core, consistent with results from linear gyrokinetic calculations indicating that ITG/TEM modes are unstable in the outer region but stable in the core. That the momentum pinch in the outer region is in general agreement with a low- k turbulence drive holds irrespective of whether the ion energy transport is neoclassical, which is the case for most H-mode plasmas, or not. This result is due to the fact that the ion neoclassical energy transport is large, and thus can dominate the low- k turbulence induced transport that is effectively reduced by large ExB shear. The neoclassical momentum diffusivity, however, is essentially zero, and thus is subdominant to whatever low- k turbulence induced transport remains, after being reduced by ExB shear. Therefore, the momentum transport in the electron-dominated plasmas of NSTX can be a better probe of the low- k turbulence than ion energy transport. A Beam Emission Spectroscopy system that will measure the turbulence spectrum at low- k is presently being installed on NSTX, and future experiments will be performed to investigate the relation of the low- k turbulence and momentum and heat transport.

This work was supported by U.S. DOE Contract No. DE-AC02-76CH03073 at the Princeton Plasma Physics Laboratory, and No. DE-FG02-99ER54523 at Columbia University.

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