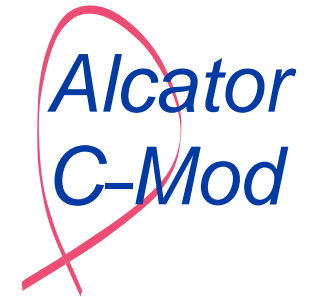


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# Operation of Alcator C-Mod with High-Z Plasma Facing Components: With and Without Boronization



- Operational experience with solid molybdenum Plasma Facing Components
- Effects of boronization
- Localized erosion of boron coatings
- Role of ICRF sheath enhancement

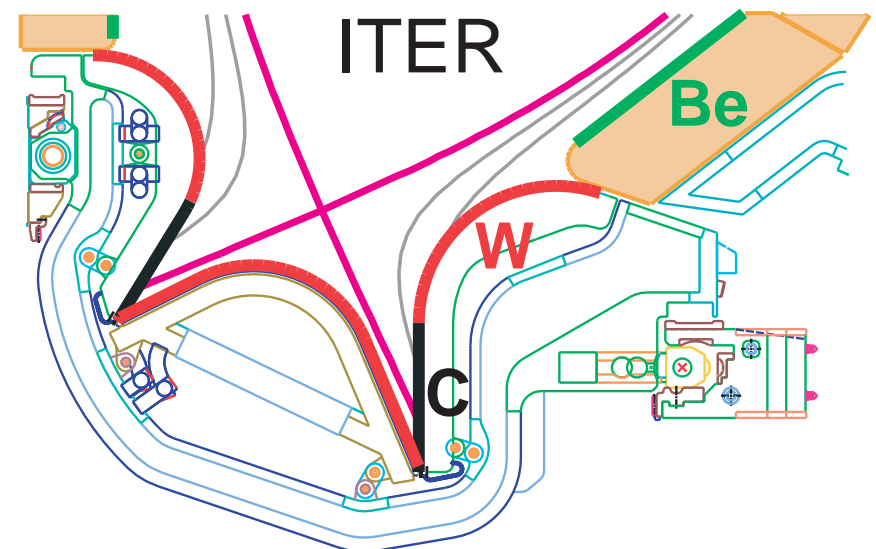
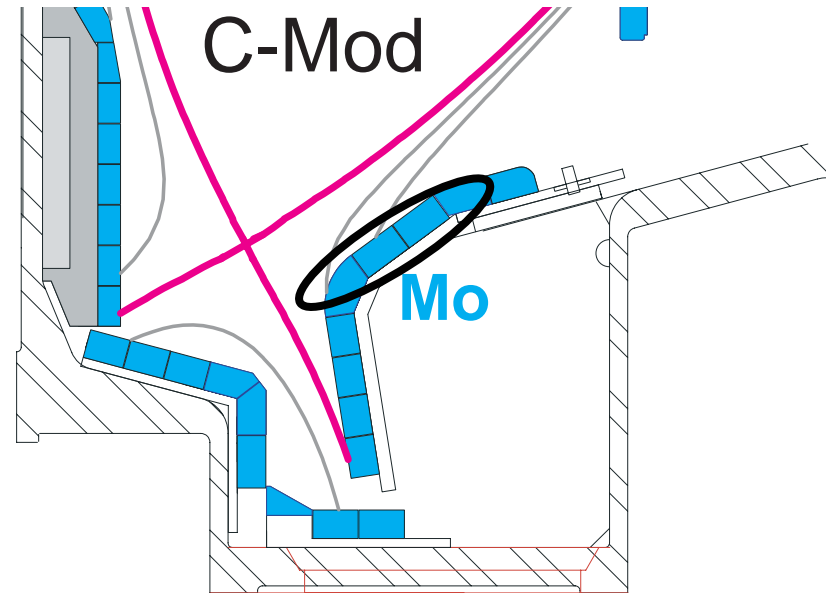
E. Marmor, Y. Lin, B. Lipschultz, D. Whyte, P. Bonoli, C. Fiore, M. Greenwald, I. Hutchinson, J. Irby, M. Reinke, J. Rice, S. Scott, J. Terry, S. Wolfe, S. Wukitch, Alcator C-Mod Team

21<sup>st</sup> IAEA  
Fusion Energy Conference  
Chengdu, China

Paper EX/3-4

# C-Mod very well Suited to Address High-Z PFC Issues

- C-Mod exclusively employs solid high-Z PFCs (Mo)
  - Mo very similar to tungsten (W)
    - Erosion
    - D retention
    - Radiation characteristics
- C-Mod divertor conditions can be equivalent to ITER
  - Power density, divertor  $n_e$ ,  $T_e$
  - SOL: opaque to neutrals, radiation
  - ICRF heating

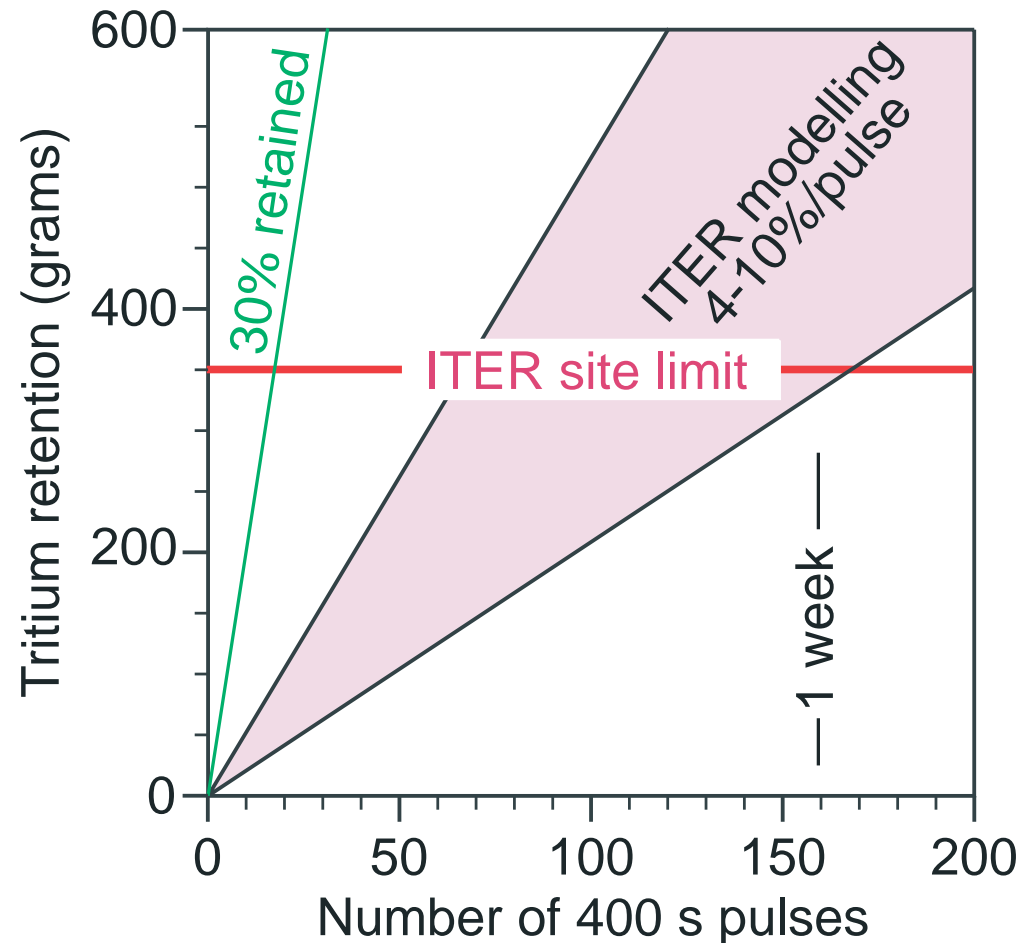


Report here on experiments comparing operation with uncoated and coated Mo

# Why Care about High-Z?

- In ITER, serious concern about tritium retention (co-deposition with carbon)
- Looking beyond ITER, to reactors, additional issues
  - Erosion at first wall
  - Neutron damage to carbon-based materials
    - Drop in thermal conductivity
    - Material swelling
- Currently ITER plans to relegate carbon to small fraction of PFC surfaces
- Multiple studies have led reactor designers to choose tungsten

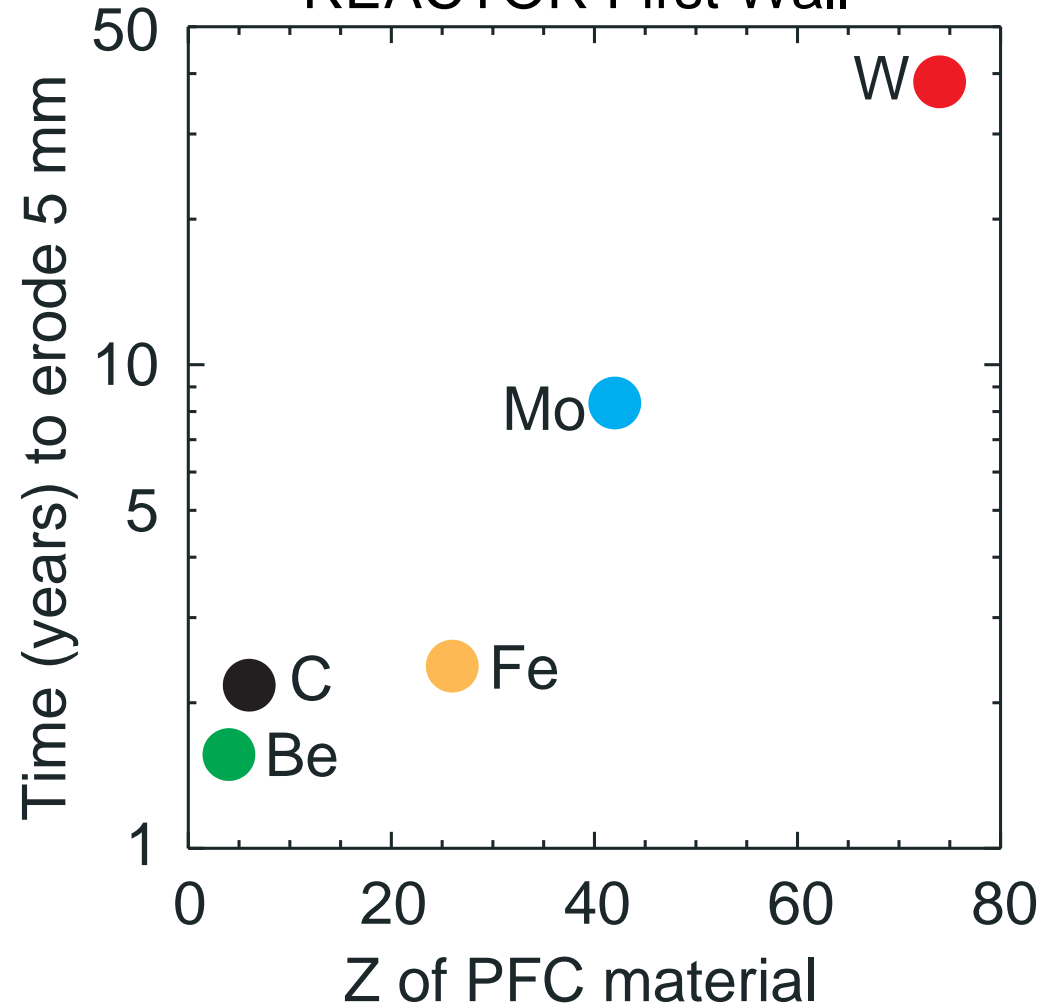
Simple extrapolation of tritium retention from JET and TFTR results (carbon PFCs)



# High-Z has Advantages; Carries Risks

- Advantages
  - Very low erosion rate
  - Reasonable resistance to neutron damage
  - Low tritium retention (at least in some non-tokamak tests)
- Risks
  - Melting leads to enhanced heat loads
  - Allowable W concentration in the core plasma is very low ( $<10^{-5}$ )
- Most of the world's divertor tokamak database developed using carbon PFCs\*

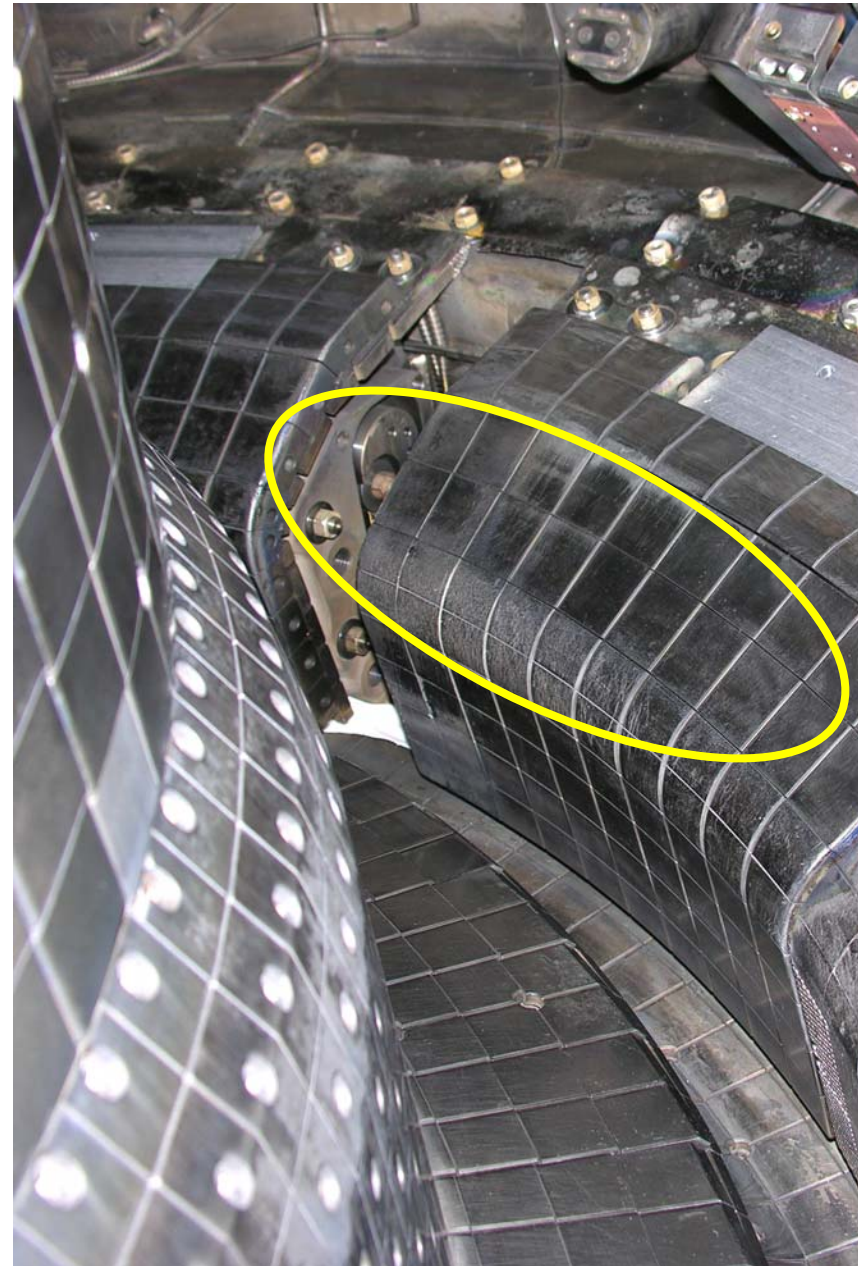
Estimates of Erosion Rates for  
REACTOR First Wall



\*ASDEX-Upgrade using W-coated graphite [Dux, previous talk]  
JET planning to use W

# Boronization used Routinely for Wall Conditioning

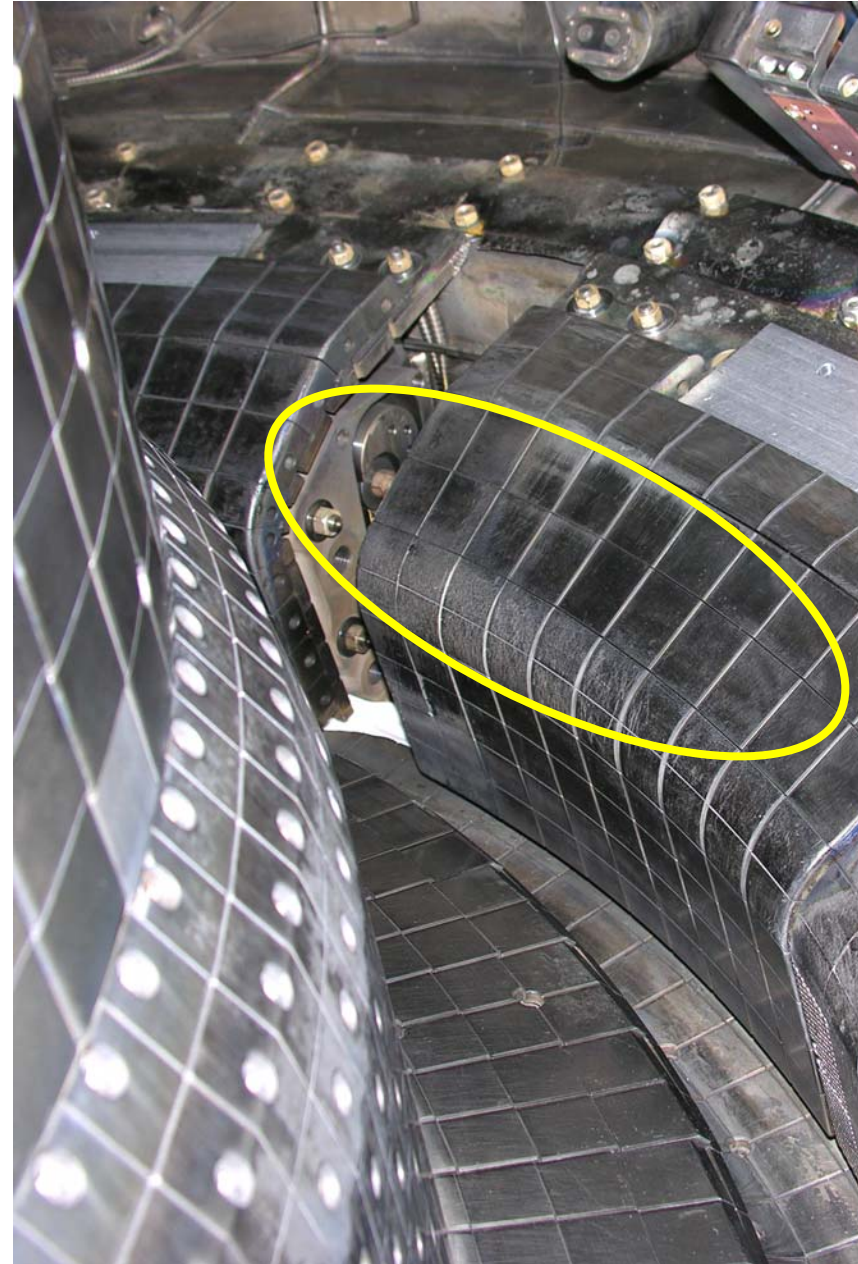
- Discharge cleaning and boronization accomplished using Electron Cyclotron resonance low temperature plasma Discharge (ECD)
- ECD parameters:
  - 2.5 kW, 2.45 GHz RF
  - Use toroidal field only; scan to move resonance from inner wall to beyond outer limiters ( $B_{\text{resonance}}=0.088$  tesla)
  - $T_e \sim 10$  eV,  $T_i < 1$  eV,  $n_e \sim 10^{16} \text{ m}^{-3}$
  - Cleaning using deuterium
  - Boronization using 10%  $\text{B}_2\text{D}_6$  / 90% He
- ~10-hour boronization
  - average coverage of 200 nano-meter





# Boronization used Routinely for Wall Conditioning

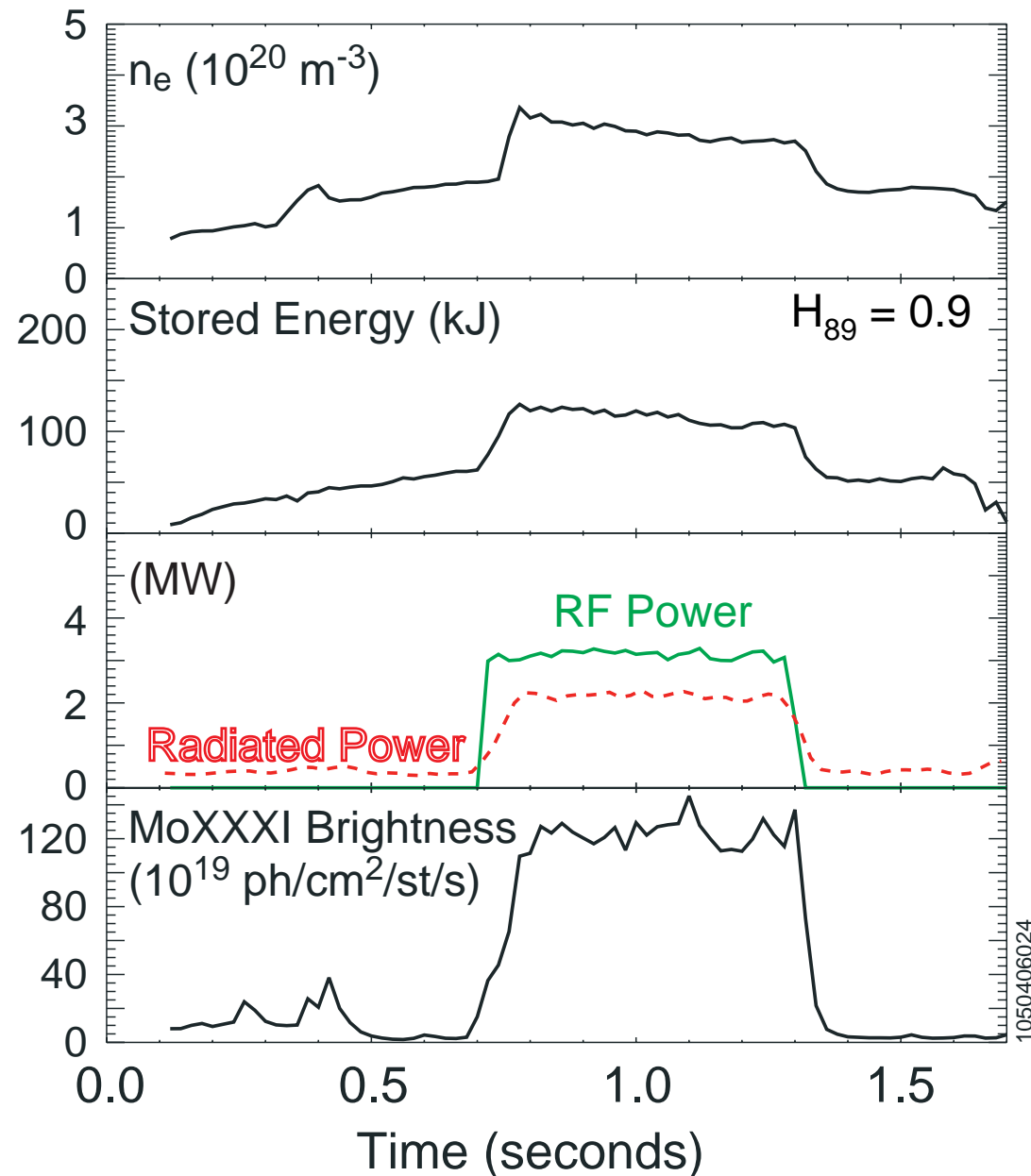
- Boronization employed almost from the start of high power ICRF operation on C-Mod
  - Relatively thick B layers built up in most areas (~6 mm)
- Secondary protection tiles (around RF antennas) utilized boron-nitride
- Prior to start of 2005 campaign
  - All PFC surfaces cleaned of B
  - BN replaced with Mo





# Clean Mo PFCs with High Power ICRF Heating Leads to High Core Radiation

- Unboronized surfaces incompatible with good energy confinement using strong ICRF auxiliary heating ( $\tau_H/\tau_{ITER89} < 1.3$ )
- In all cases Mo radiation from the confined plasma rises rapidly, cooling the plasma
  - Pedestal pressure is suppressed
- Various approaches to improve not effective
  - Increase outer gaps
  - Force divertor detachment
  - Cool SOL with D<sub>2</sub> gas
  - Li pellet wall conditioning
  - Boron dust injection

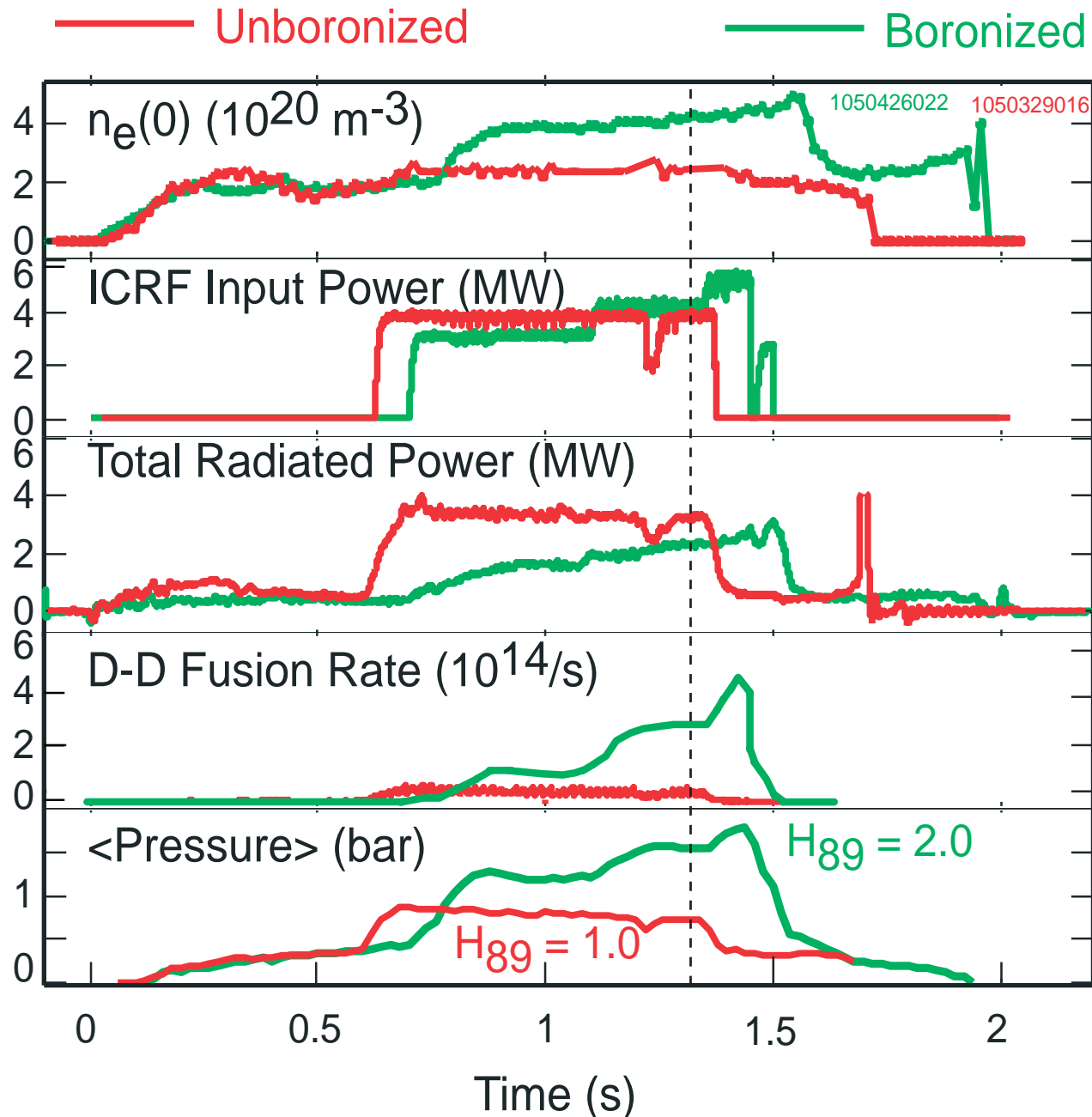




# Boronization Required for High Performance Until Recently, Performed Overnight

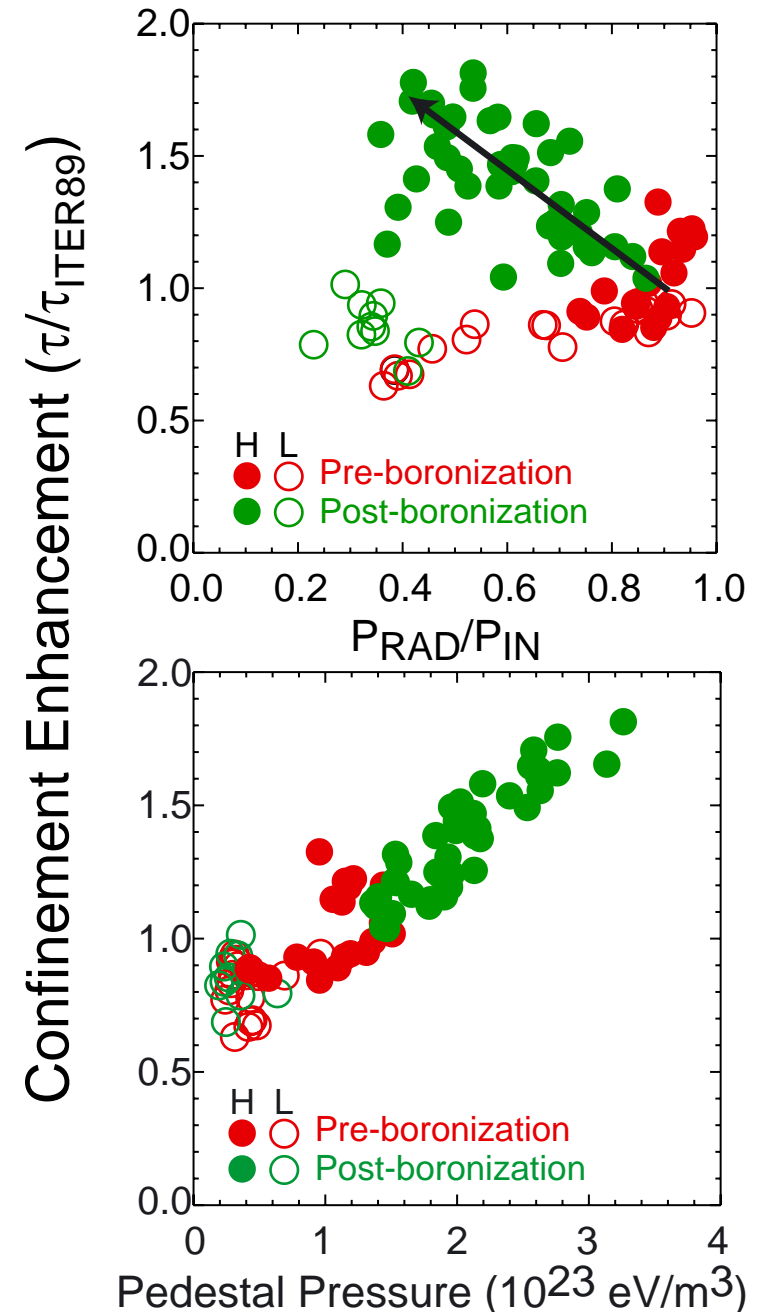


- After ECD boronization, Mo radiation reduced by factor of 5 or more
  - Fe also reduced
  - B increases
- Energy confinement ~doubles



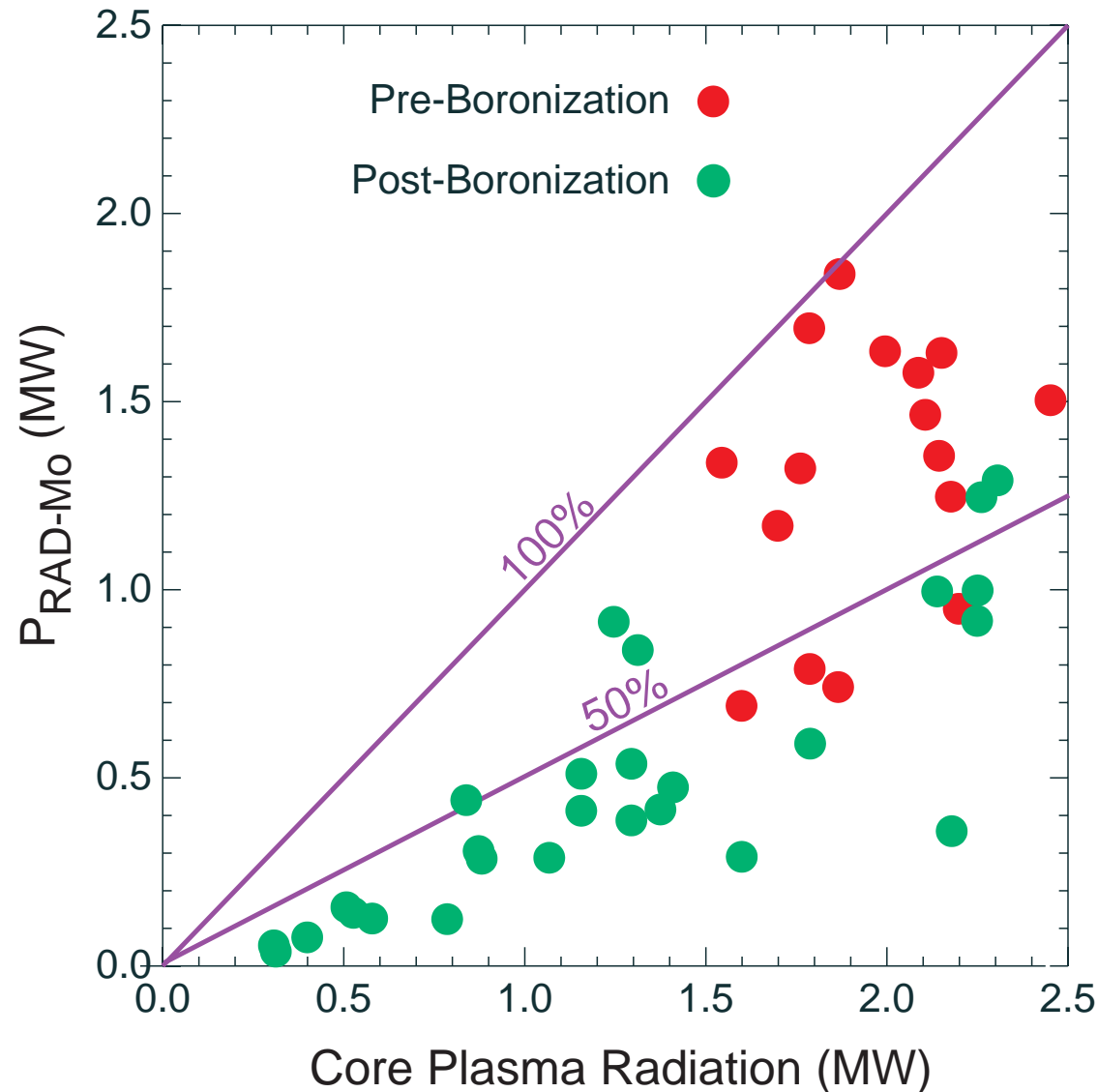
# Monotonic Improvement with Reduced Radiation

- Radiation cools the pedestal
  - Reduced pedestal pressure
  - Profile stiffness leads to decreased core temperatures and pressures
  - Mo radiates inward of the pedestal, B mostly outside
- After each overnight boronization:
  - Radiation fractions drop
  - Energy confinement in H-Mode improves



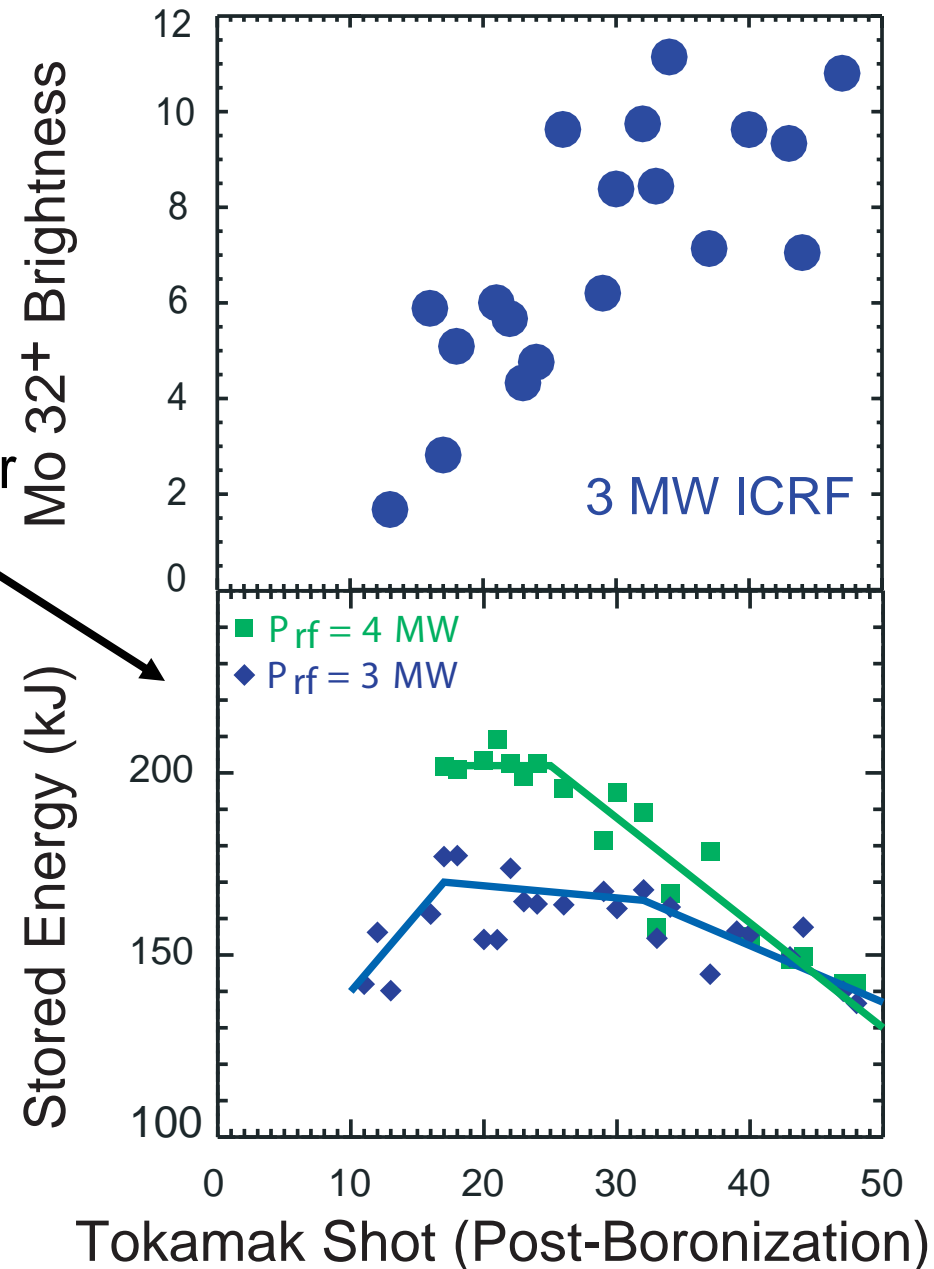
# Molybdenum is the Primary Radiator Prior to Boronization

- Prior to boronization
  - Mo accounts for majority of the radiated power
  - Fe accounts for much of the rest (~15%)
- After boronization
  - Mo radiation fraction  $\frac{1}{4}$  to  $\frac{1}{2}$  of the reduced total
  - Fe is very small (~4%)
  - B and F account for the rest



# Benefits of Overnight Boronization (10 hours) Last 20 to 50 Discharges

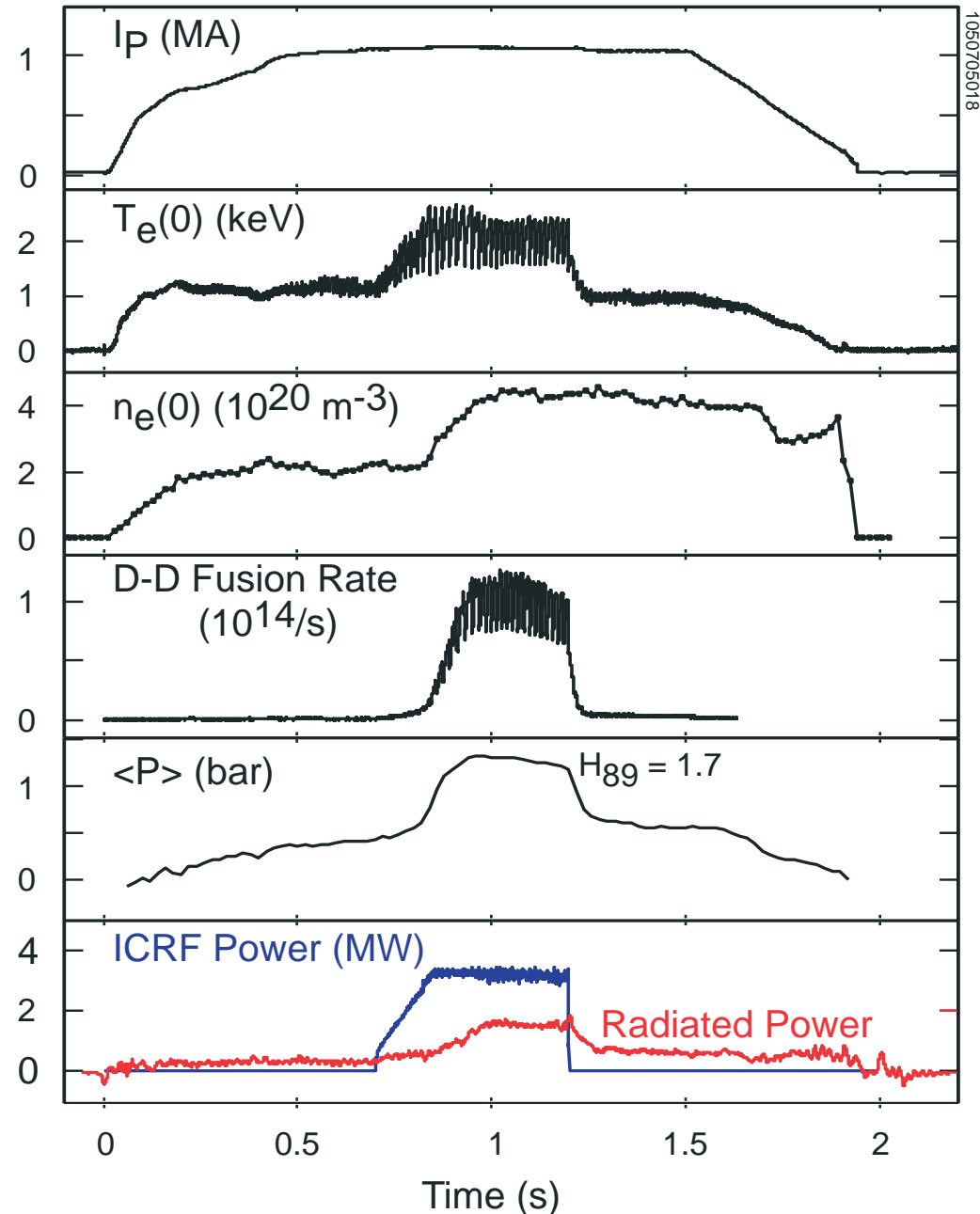
- Following overnight boronization, extended run day to examine evolution with plasma discharges
  - Mo levels rise monotonically from shot to shot
    - Fe does not increase
  - Confinement decrease apparent after ~20 high power discharges (~50 MJ total input energy)
- Post-campaign tile analysis shows thick boron layers on most tiles
  - Exceptions
    - Outer divertor, near usual strike point
    - **Top of outboard divertor, especially at leading edges**



# Inter-Shot Boronization Works Well

## Effects Persist for ~ 1 Discharge

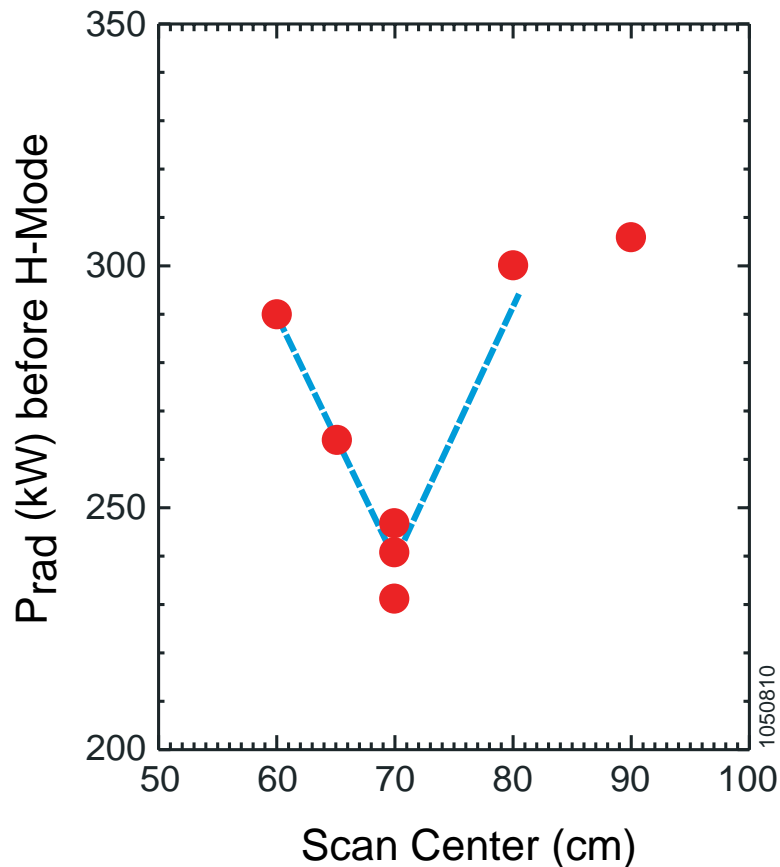
- Close to best performance recovered for discharge following 30 minutes of EC between-shot boronization
  - Localized boron coverage ~100 nanometer
- Effect wears off after 1 to 2 discharges
  - Opportunity to study and try to optimize parameters





# EC Resonance Position Affects Efficacy

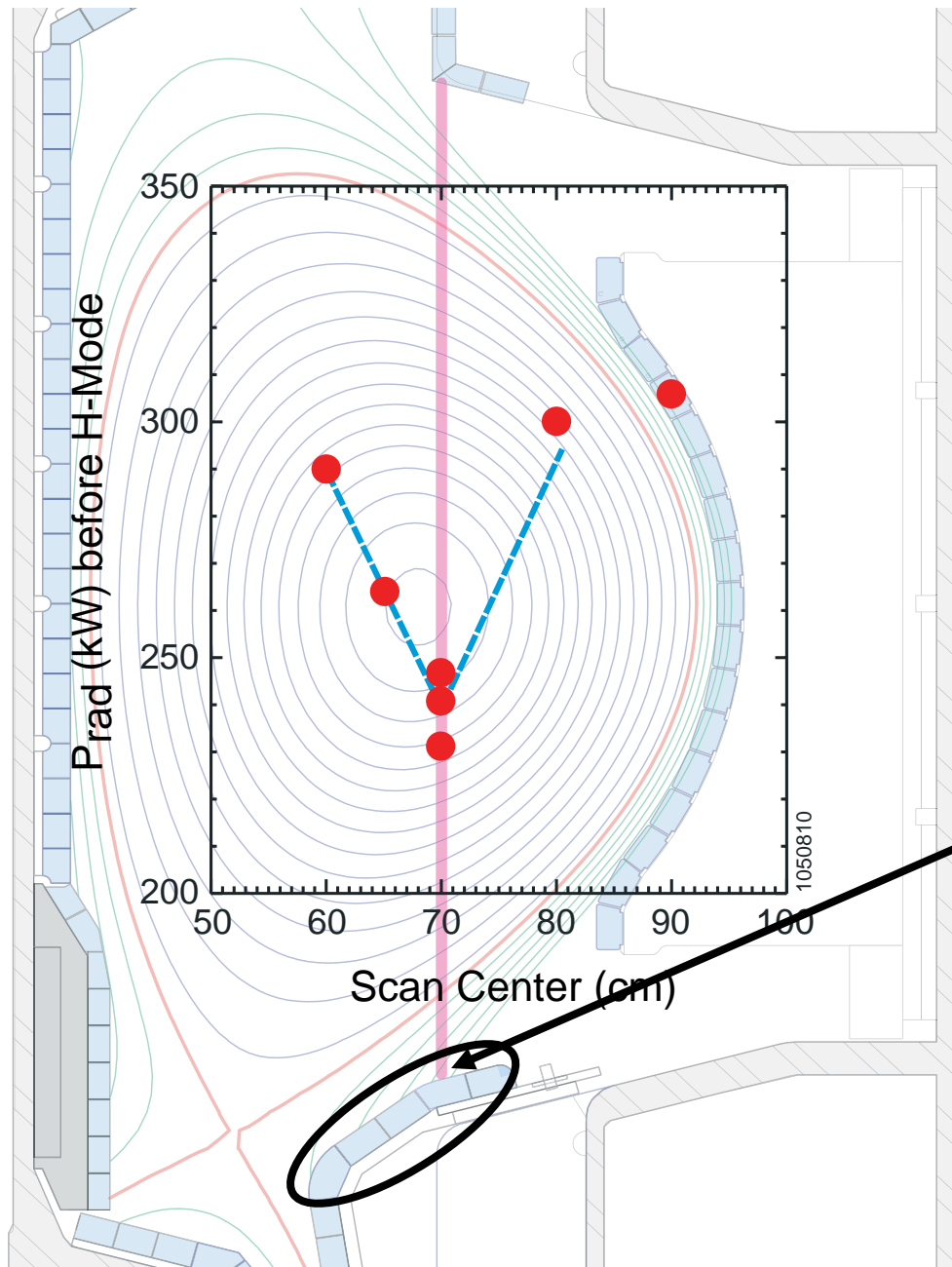
Resonance scanned  $\pm 5$  cm



- Plasma breakdown at EC resonance (cylinder at fixed R)
  - plasma unconfined to larger R
- Clear result that some locations are better than others

# EC Resonance Position Affects Efficacy

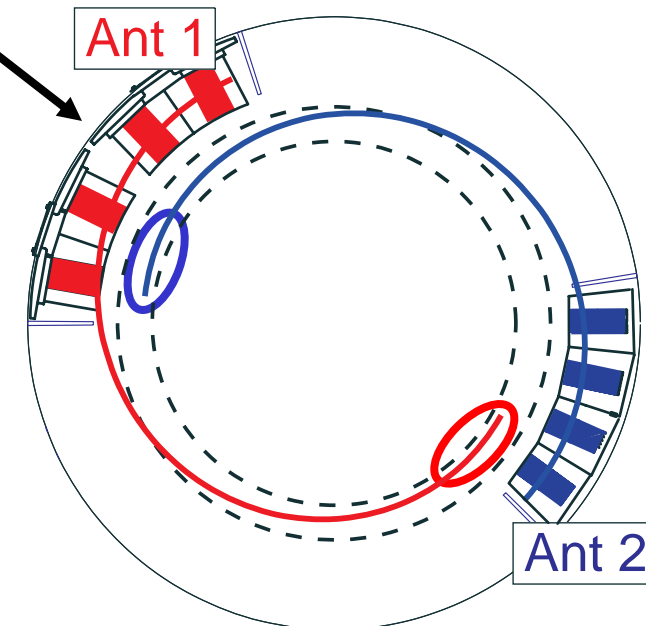
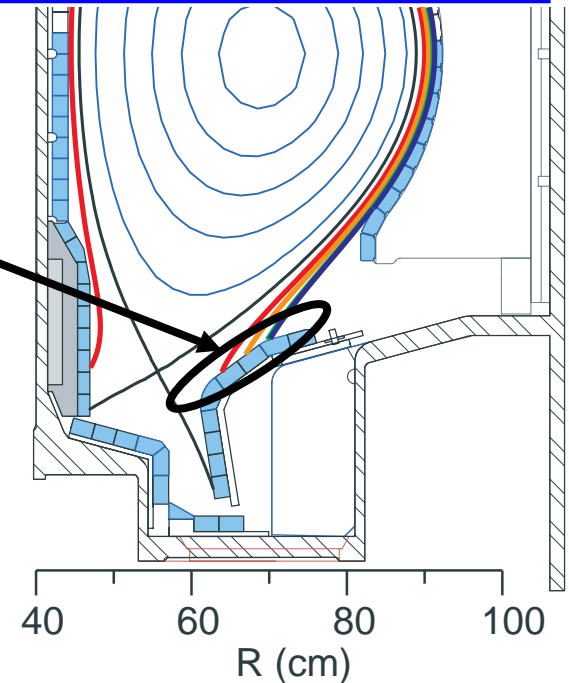
## Appears to optimize near top of outer divertor



- Plasma breakdown at EC resonance (cylinder at fixed R)
  - plasma unconfined to larger R
- Clear result that some locations are better than others
  - Maps to top of outer divertor
  - Away from highest heat-flux region near strike point

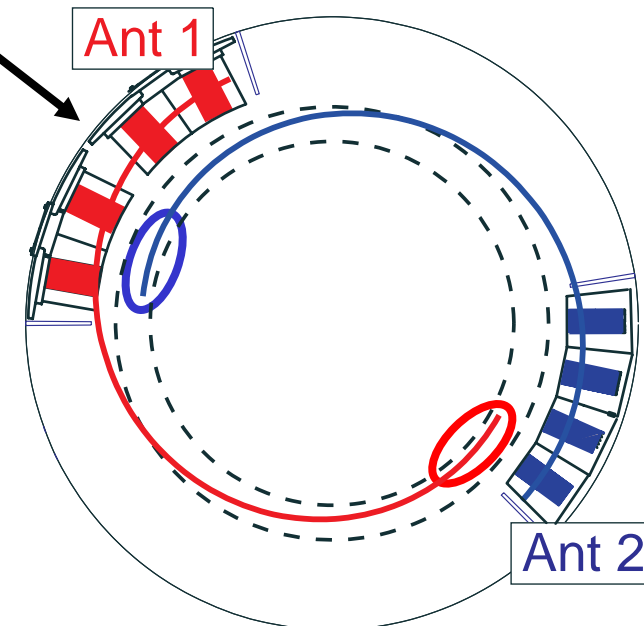
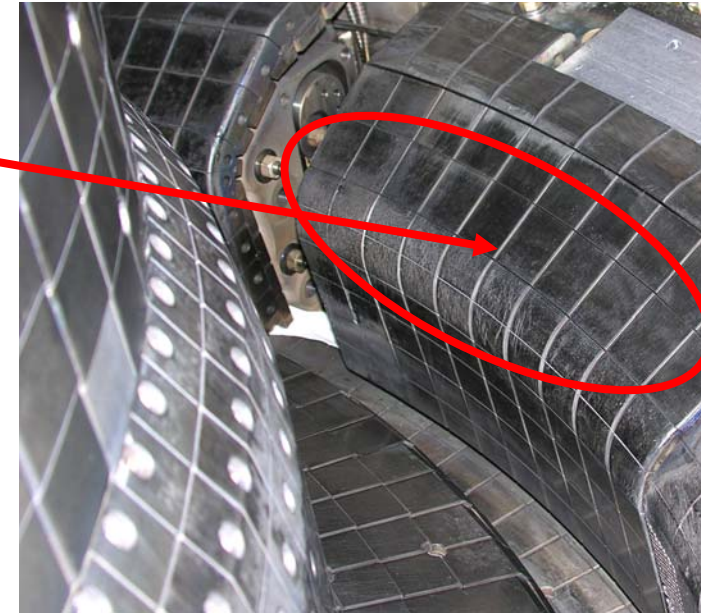
# ICRF Sheath Enhancement Responsible for Boron Erosion

- For the equilibrium studied, field lines map from the ICRF antennas to top of outer divertor
- RF sheath-potential enhancement can lead to increased sputtering
- Antennas are on opposite sides of the torus
  - Corresponding field line mapping is to toroidally distinct regions at top of outer divertor
- Conjecture: Boron is preferentially eroded in area with enhanced sheath
  - Supported by energizing different antennas on alternate shots
  - RF erodes boron at least 5 times as fast as ohmic (per joule)
  - Direct sheath potential probe measurements confirm ( $V_{\text{sheath}} > 100\text{V}$ ) [Wukitch, FT/1-6]



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# Summary

- 
- High power heating with high-Z metal PFCs on C-Mod yields high core radiated power and reduced performance
  - ECD boronization is very effective in reducing radiation from high-Z impurities (Mo)
    - Leads to dramatic performance improvements
  - Between-shot boronization effective (~1 tokamak discharge)
    - Investigation into erosion localization, Mo sources
      - High heat-flux region of the divertor, near strike points, is not the critical source region for Mo which reaches the core plasma
      - ICRF sheath enhancement (>100 V) at top of outboard divertor implicated
  - Possible implications for ITER
    - Tritium retention in high-Z PFCs?
    - Compatibility of tungsten?
    - Wall-coating/conditioning for long-pulse?