"Assessment of Compatibility of a System with Fast Reactors with Sustainability Requirements and Paths for Its Development"

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Historical Perspective

- Joint Assessment Study on INS based on CNFC-FR was initiated by Russian Federation in Oct 2004 as an integral part of INPRO Phase I activities on assessment of INS with use of INPRO methodology
- The Study was implemented by Canada, China, France, India, Japan, Republic of Korea, Russian Federation, and Ukraine
- It is completed in November 2007
- Final Report published as an IAEA-TECDOC





MS - Participants of the JS



Arrangement of Joint Study

National Level

- Formation of national teams
- Representations in the JS Scientific & Technical Committee (STC)
- Formation of working groups on specific topics

IAEA

- INPRO IG, INPRO Task Managers
- IAEA experts

Main documents for guidance

- Terms of Reference and concept of the study
- Roadmap
- INPRO Methodology and Manuals on assessment of compatibility of an nuclear energy system with sustainability requirements

Main Objectives of the Joint Study were to:

- Determine milestones for the INS CNFC-FR deployment at national and global levels
- Assess the INS CNFC-FR for satisfying the INPRO sustainability requirements and identify areas for improving the INS with R&D or/and institutional measures
- Provide feedback on the INPRO methodology improving

Milestones for the INS CNFC-FR deployment at national and global levels





National Strategies and Scenarios



- Various national expectations in nuclear growth: from about zero up to 'as much as possible'
- Promising global market for INS CNFC-FR: 300-500 GW by 2050
- Different level of the technology development: from research reactors up to pilot commercial NPPs and associated fuel cycle
- Specific focus on the INS role: fuel assurance or waste reduction

Selection and Evaluation of Technologies



FR related national programms

| | Coolo | FP type | | | | | |
|----------------------|---------------------|------------------|------------------------|----------------|--|--|--|
| Country | nt | Experiment al | Prototype | Commerci al | Fuel | Process | |
| Russia | Na, Pb, Pb-Bi | 1969, 1973 | 1980, 2012- 2015 | 2020202 5 | UO ₂ , MOX, nitride improved PURE pyroprocess, vit packing | | |
| Japan | Na | 1977, 1994 | 2020 | 2050 | MOX, metal | improved PUREX, pyroprocess, injection molding | |
| France | Na, gas | 1967, 1973 | 2020+ | 2030+ | MOX, nitride | improved PUREX, pyroprocess | |
| USA | Na (Pb) | 1964, 1980 | 2014 | 2023 | MOX, metal | pyroprocess, injection molding | |
| China | Na | 2008 | 202020 25 | 2030203 5 | UO ₂ , MOX, metal | pyroprocess, injection molding | |
| India | Na | (1984) | 2010 | 2020 | MOX, metal | improved PUREX, pyroprocess | |
| Republic of Korea | Na | | 2030 | | Metal | pyroprocess | |

Commonalities among National Scenarios

- Joint scenario: U constraint would be evident by 2050 as well as the problem of SF accumulation in OTFC
- High emphasis on safety, economics, waste, proliferation resistance to meet sustainability requirements
- > A small serial of SFRs are to be deployed in 15-20 years
- Generic features of the near/medium CNFC-FR: sodium, pool type, pellet MOX fuel, advanced aqueous reprocessing
- Commonalities provides a basis for building a 'reference model' of near/medium INS CNFC-FR

Variants among National Scenarios

- Times of introduction of FRs
- Deviations in Capacity, Reactor Type (loop Japan)
- Vision of a robust INS CNFC-FR: variations in coolant, design arrangement (2-circute), fuel options, breeding

Assessment of the INS CNFC-FR for satisfying the INPRO requirements and identifying areas for its improving with R&D or/and institutional measures





INS assessment in INPRO: bottoms up approach

A criterion (CR) is fulfilled when its indicators comply with Accep. Limits

A user requirement (UR) is met when all its CR are fulfilled

A basic principle (BP) is achieved when all its URs are met



If a criterion is not fulfilled then improving should be addressed including R&D and/or institutional measures

Safety: philosophy and means

• SFR operational experience as a basis for safety assessment:

- evidence to meet requirements of the current regulatory basis;
- occupational and public exposure at the best level in NP

• 4 INPRO safety Basic Principles (BPs) based on the IAEA Principles, utility requirements & an extrapolation of current trends:

- an enhanced application of the concept of defence in depth (DID);

- increased use of passive systems and inherent safety features;

- reduction the risk due to radiation exposure to the level of risks arising from facilities of other industries with a similar purpose;

- sufficient level of R&D to be performed for new nuclear designs.

• Examples of SFR inherent safety features are:

- negative reactivity feedback in power and temperature disturbances, stability of neutron pattern, no poisoning effects;

- the excellent heat transfer and high boiling point of sodium resulting in a very low pressure and low stored energy of the coolant.

• An example of engineered safety features:

- the installation of double walled pipes and vessels to avoid sodium leaks.



Safety of upcoming INS CNFC-SFR: specific findings

- Specific findings on upcoming INS CNFC-SFR:
 - probability for radioactivity release to public domain <10⁻⁷ per unit year;

- disadvantages of sodium due to its water/air reactions are minimized with adequate design solutions; sodium leakage and fires do not make a critical input into overall safety of FR

• R&D programs for the INS CNFC-SFR:

- core physics & technology design related to safety;
- preventive surveillance and inspection aspects;
- development of sensors and repair of welds in sodium;
- development and validation of sodium fire models and PSA analysis, etc.

Overall INPRO evaluation gives every reason for a high safety judgement on the INS CNFC-SFR provided that identified R&D are carried out

In spite of high appraisal of the CNFC-SFR safety, the need for development of alternative concepts was clearly stated due to crucial role of CNFC-FR for the NP future.

Safety: long-term expectations & perspectives

Long-range R&D for safety of alternative options:

- lead-bismuth, lead, gas and steam cooled systems;
- SMR of modular type;
- ADS, MSR;
- advanced MOX, metal, nitride and other innovative fuels

Perspective INS CNFC-FR should to provide ability to avoid, in case of a severe accident, a need for relocation or evacuation measures outside the plant site, apart from those generic emergency measures developed for any industrial facility used for similar purpose

Excel safety the conceptual INS CNFC-FR designs should be demonstrated together with technical feasibility, reliability, and improved economics



Environment: impact (1/2)

Two Basic Principles are developed in the area:

- BP1 calls for acceptability of environmental impacts on humans & environment;
 BP2 requires the confirmation of the long term availability and optimized use of material resources needed to operate a nuclear system
- To assess the CNFC-FR impact via releases of toxic elements and depletion of resources, 6 NESs were analyzed
- Emissions (t/TWhe) received in a life cycle analysis (LCA) using TEAM software for NFC of France for two NESs:

| Emissions | GHG eq. CO2 | CO2 | CH4 | N2O | NOx | SOx | Particles |
|-----------|-------------|------|-----|------|-----|-----|-----------|
| OTFC-PWR | 4600 | 4400 | 6 | 0.1 | 15 | 24 | 41 |
| CNFC-FR | 1400 | 1400 | 2 | 0.02 | 4 | 7 | 2 |

The emissions are mainly from the fabrication process of the plants and components and not to the electricity production

LCA for coal and natural gas FC: ~10⁶ & ~4.6·10⁵ t/TWhe CO2 eq., respectively

The study confirms that NP in the whole, and CNFC-FR especially, are very effective options for GHG emissions reduction



Environment: impact (2/2)

Public exposure (mSv/yr) of OTFC-PWR & CNFC-FR provided by LCA (dose

values are based on experience with existing NFC of France):

| | Minin/ Mill. ¹ | Conver. | Enrich. | Fabric. | NPP | Reproc. | Storag. | HLW dispos. ² |
|--------------|------------------------------|--|---------------------|------------------------------|-------------------|------------------|---------------------|-----------------------------|
| OTFC -PWR | <1 | 2·10 ⁻³ - 7·10 ⁻² | ~3·10 ⁻⁴ | ~6·10 ⁻⁴ (UOX) | ~10 ⁻³ | | ~2·10 ⁻² | < 2.5·10 ⁻¹ |
| CNFC-FR | | | | ~10 ⁻⁵ (MOX) | ~10 ⁻³ | 10 ⁻² | ~2·10 ⁻² | < 2.5·10 ⁻¹ |

¹ Main contribution; ² Safety limit

Both OTFC-PWR and CNFC-FR produce a public radiation dose far below the current regulatory limit of 1 mSv/yr for public exposure



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Environment: resource consumption

Table below presents results of LCA for the consumption (t/TWhe) of natural uranium, oil, gas and coal for OTFC-PWR and CNFC-FR

| | Oil | Coal | Natural gas | Natural uranium |
|----------|-----|------|-------------|-----------------|
| OTFC-PWR | 600 | 1000 | 400 | 23.4 |
| CNFC-FR | 100 | 400 | 100 | 1.7 |

A CNFC-FR provides significant reduction of resource depletion (especially natural uranium) in comparison to a OTFC-PWR

In both cycles, oil, coal, and natural gas consumption relates mainly to the fabrication process of the plants and is hundreds times less than for fossil sources

The environmental effects for CNFC-FR are well within the performance envelope

Having a potential to ensure natural resource for hundreds years of operation, INS CNFC-FR is practically a renewable option

Waste: minimization of waste generation

Starting point:

- The CNFC-FR system has practically showed its potential to meet all today requirements related to waste management
- Conditioning of waste from Pu recycling is industrial reality today and a practical milestone in reaching ultimate goals of the closed cycle strategy
- A neutronic calculation (COSI code) was performed to determine waste management parameters

| Waste | Pu+Am +Cm ¹ | SF assembl | SF assembl | Vitr. waste canisters | Waste to inter. stor. | Waste to final dispos. |
|-----------|---------------------------|---------------|---------------|--------------------------|-----------------------|------------------------|
| Unit/TWhe | kg | number | m³ | number | m ³ | m ³ |
| OTFC-PWR | 27.9 | ~1 | 5 | - | - | - |
| CNFC-FR | 0.15 | - | - | 1.49 | 0.26 | 0.59 |

•In CNFC-FR versus OTFC-PWR Pu/Am/Cm to final disposal is ~200 times less (Np is not considered due to low contribution on radiotoxicity and thermal loading)

•Achieving a 90% reduction of fuel waste requiring repository disposal and a cut off 5-10 times repositories number

•Recycling & reuse of low radioactive materials from decommissioning should be included into further assessments

Waste: avoidance of burdens on future generations

In OTFC, SF needs several 100000 years to reach an equivalent level of radiotoxicity as natural U ore

Recycling of all actinides helps to meet the criterion in several 100 years (Fig)

The assessors claim that the technology of waste immobilization will be available on an industrial scale on time to achieve a safe end state of nuclear waste of a CNFC-FR



CNFC-FR has a very high potential in waste management area and it is one of the main drivers for the INS introduction in near future

Proliferation Resistance

• PR intrinsic features:

- flexibility in fissile materials management (avoid Pu "mines");
- exclusion of Pu separation in reprocessing;
- possibility to seal a reactor core for fuel campaigns of 10-20 years;
- limiting fuel transport by co-location of FC facilities & reactors
- In spite of wide spectrum of intrinsic features, there is no technical solution of the non-proliferation problem
- Extrinsic measures & intrinsic features both are essential for PR, while the latter may play a complementary role

PR of the CNFC-FR due to realization of the intrinsic features could be comparable or higher than PR of OTFC

The INS provides key technologies for balanced use of fissile materials and has a high potential for applying extrinsic institutional arrangements ensuring elimination of their worldwide accumulation



- Information to public: risk communication approach
- Professional perspectives:

- not sufficient inflow of young scientists & engineers who could provide a long-term perspective

CNFC-FR - a key element for multinational approach to the NFC since it provides possibility to take back SF as a valuable fuel resource

CNFC-FR: high potential to enhance sustainability of national/global NP by utilizing synergy between TR & FR as well as between countries (Fig) with different approaches to NFC

Economics: to meet Acceptance Limit

- Factors providing potential of the CNFC-FR competitiveness:
 - competitive energy cost not sensitive to U market price;
 - U price will partly justify introduction of CNFC-FR, even \$/KWe is higher of TR;
 - probable increase of the cost for TR spent fuel disposal.
- Technical feasibility is demonstrated with electricity cost rather near to the area of competitiveness
- The investments required to design, construct, and commission of the CNFC-FR can be raised mainly by the MS with a large nuclear program

Economics of CNFC-FR is an area that needs further RD&D for demonstration compatibility with INPRO requirements

Emphases on R&D reducing capital cost & investments





Economics: to meet Acceptance Limit



R&D for the near/medium CNFC-FR:

- material reduction: less N of loops, thickness of vessels; improving and cut of auxiliary systems, efficient radiation shield; reduction of building volumes;
- a serial FR: BN (Russia), FBR (India), JSFR (Japan); reduction of a construction time;
- improving thermodynamics & thermo-efficiency; increasing of burn up; life time extension

R&D for radically innovated system:

- two-circuit concepts, gas turbine, high temperature designs

Economics: diversity in national conditions & commonality in tasks

| | | CR1.1 Cost of Energy | |
|-------------------|---|---|--------------------------------------|
| | Indi | | |
| JS Participant | IN1.1.1 Cost of nuclear energy, C _N mills/kWh | IN1.1.2 Cost of energy from alternative source, C _A mills/kWh | Acceptance Limit Value AL1.1.1 |
| France | 35.4 | 44.0 | 44.0 |
| India | 41.0 | 63.0 - 73.0 | 45.0 |
| Japan | 15.1 | 26.6 | 26.6 |
| Korea | 31.2 | 26.0 - 34.0 | 34.0 |
| Russia | 17.7 | 24.5 | 24.5 |

CNFC-SFR: will be affordable in 15-25 years in countries mastering the technology

Beyond 15-25 years: expectation for further economic improvement through identified RD&D and expected industrial deployment

Overall judgment on INS CNFC-FR

- INPRO joint evaluation of the CNFC-FR demonstrates high potential of the system to enhancing sustainable features to NP
- Introduction of the SFR on the basis of synergy with TR will help to solve practical problems of MS, like SF & Pu accumulation or lack of fissile material resource
- In a global perspective, the INS could become a key element for multilateral approach to NFC providing possibility to take back SF as a valuable fuel resource
- R&D are identified as a main condition for commercial introduction of the INS



Collaborative Project Proposals Initiated/Discussed at the JS Meetings

| N | INPRO Collaborative Project |
|---|--|
| 1 | A Global Architecture of INS based on Thermal and Fast Reactors with the inclusion of Closed Nuclear Fuel Cycle (GAINS) |
| 2 | Assessment of advanced and innovative nuclear fuel cycles within large scale NES based on CNFC concept to satisfy principles of sustainability in the 21st century (FINITE); |
| 3 | Integrated Approach for the design of Safety Grade Decay Heat Removal System for Liquid Metal Reactor (LMR) |
| 4 | Investigation of technological challenges related to the removal of heat by liquid metal and molten salt coolants from reactor cores operating at high temperatures (COOL) |



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Feedback to the INPRO methodology





Feedback to INPRO methodology (1/2)

INPRO methodology was found to be a useful guide for the team of qualified experts in assessing a large innovative system of NP

- Some findings in the CNFC-FR evaluation that were made due to prescriptions of INPRO method to take into account:
 - the whole life time of the INS (cradle to grave) and all its components;
 - level of maturity of the INS;
 - all subject areas of the INPRO sustainability concept;
 - possible synergy between nuclear systems, not only competitiveness.
- Proposals for the methodology/manual improvement:
 - to cut uncertainty of AL values and related ambiguity of judgment by arriving at a kind of International Standards or consensus;

 to supplement ALs with a target value scale for defining a prospect for INS development for long periods



Feedback to INPRO methodology (2/2)

- Perspective directions of the methodology and manual development:
 - increasing of discriminative power of the method;
 - developing an agreed approach for consolidation results received in different topical areas;
 - extension of analysis from nuclear to all energy sector for better understanding capability of INS to give a global answer to global energy challenges

Further improvement of INPRO methodology is a challenging task which will require time and resource

Yet, there are important tasks where the developed INPRO method, based on assessing long-term perspective with an aim to define practical steps to be done today, will be very helpful

OUTCOMES

- Innovative and unique multinational organizational structure providing framework for future studies was created
- The assessment helped to tune national strategies to a global framework and discover specific features of the CNFC-FR which, being realized, would ensure higher level of NP sustainability
- Multinational input to improving of the INPRO methodology was provided
- Several INPRO CPs on institutional and technological measures for the system improvement were initiated
- Interactions and understanding with relevant groups (OECD/NEA, Red-Impact project)



... Thank you for your attention

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