



*International Conference on Fast Reactors and Related Fuel  
Cycles (FR09) - Challenges and Opportunities -*

**RESULTS OF POST-IRRADIATION EXAMINATION  
OF INERT MATRIX FUEL COMPOSITIONS  
IRRADIATED IN BOR-60 REACTOR UP TO 19 at.%  
UNDER THE RUSSIAN-FRENCH EXPERIMENT  
BORA-BORA**

*F.N. Kryukov, S.V. Kuzmin, **O.N. Nikitin,**  
L.M. Zabudko, I.S. Kurina,  
B.D. Rogozkin, Yu.A. Ivanov,  
J. Noirof, S. Bejaoui*

## *In the frame of the MINATOM (Russia) - CEA (France) collaboration...*

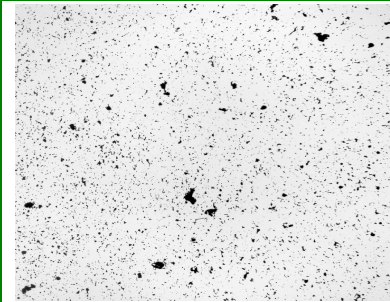
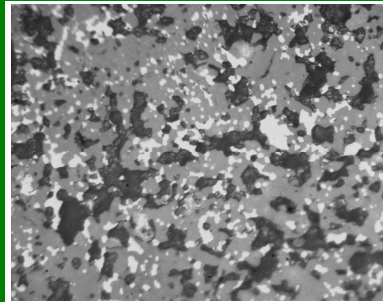
*Selection and development of techniques to produce inert matrix fuel compositions and research of their radiation resistance are actual for use of plutonium and transmutation of minor-actinides.*

*In the frame of the MINATOM (Russia) - CEA (France) Agreement on fast sodium reactors experiment BORA-BORA was performed to investigate different fuel compositions with high plutonium content under the BOR-60 conditions. Among them:*

**Fuel composition 1: 40% PuN+60% ZrN – 2 fuel pins.**

**Fuel composition 2: 40% PuO<sub>2</sub> + 60% MgO – 2 fuel pins.**

## *Manufacturing characteristics*

Fuel composition	40%PuN+60%ZrN	40%PuO <sub>2</sub> +60%MgO
Pin number	7R, 9R	13PI, 23PI
Pellet density (%Dth)	84	89
Pellet diam. (mm)	5.9	5.9
Diam. Gap (μm)	200	200
Grain size (μm)	25-30	1-10
Fissile stack length (mm)	450	450
Crystalline lattice parameter, nm and Phase composition	0.476 – Pu(Zr)N; 0.470 – Zr(Pu)N Solid solutions (differing ratio Pu/Zr)	0.5395 – PuO <sub>2</sub> 0.5198–Pu <sub>2</sub> O <sub>3</sub> ·2PuO <sub>2</sub> 0.4211 – MgO PuO <sub>2</sub> +Pu <sub>2</sub> O <sub>3</sub> +MgO Mechanical mixture
Microstructure		

## *Irradiation data*

<i>Fuel pin</i>	<i>Fuel composition</i>	<i>Irradiation time, day</i>	<i>Max. burn up (at. %)</i>	<i>Linear power (kW/m)</i>	<i>Max. clad. temperature, °C</i>	<i>Damage dose in steel, dpa</i>
9R	<b>40%PuN +60%ZrN</b>	514	11.3	20.7	547	23
7R	<b>40%PuN +60%ZrN</b>	900	19.4	20.7	543	43
23PI	<b>40%PuO<sub>2</sub> +60%MgO</b>	514	11.1	10.0	551	23
13PI	<b>40%PuO<sub>2</sub> +60%MgO</b>	900	19.0	10.0	514	43

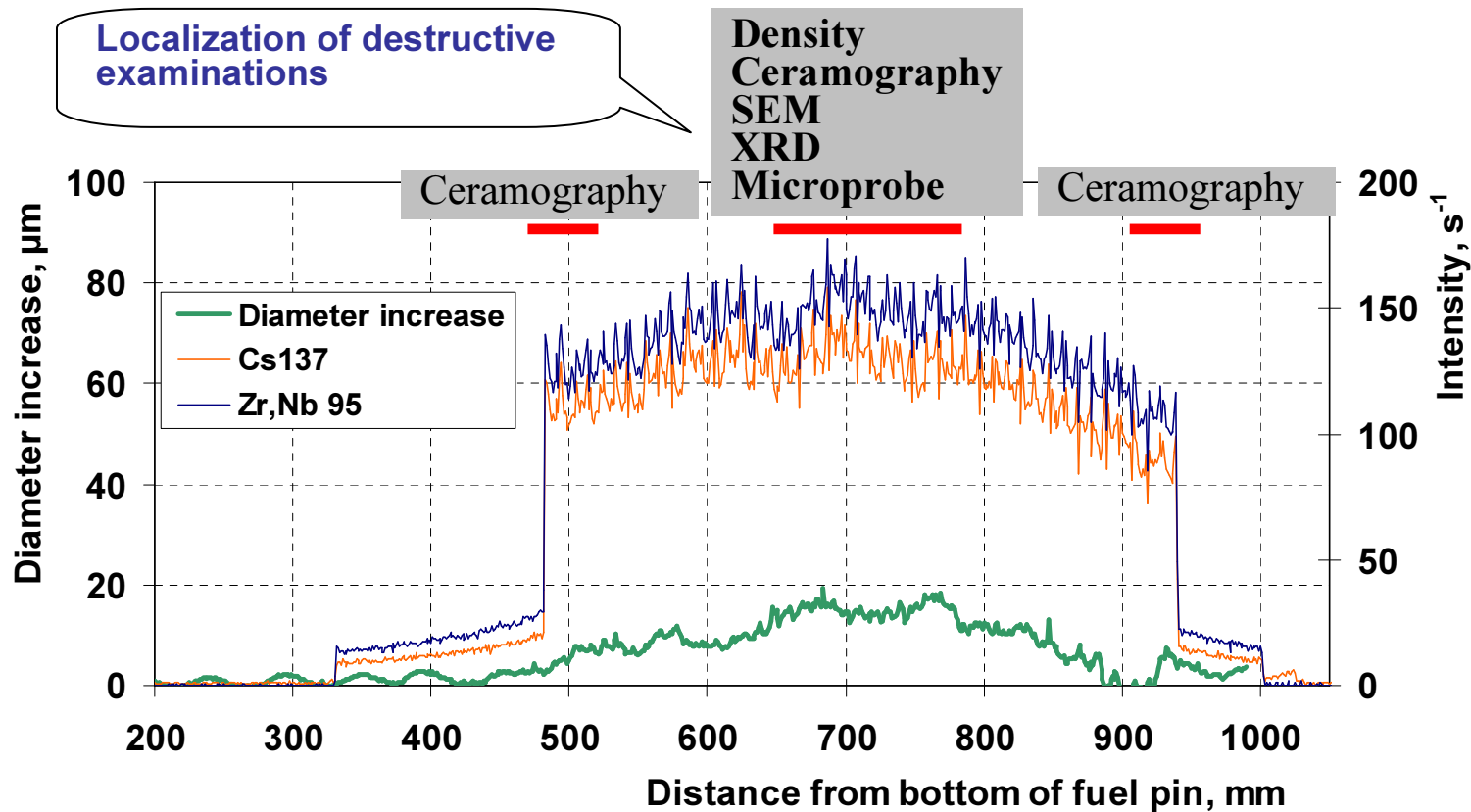


# Post-Irradiation Examinations

(fuel pin with 40% PuN+60% ZrN, burn up 19.4 h.a. %)

## Non Destructive Testing:

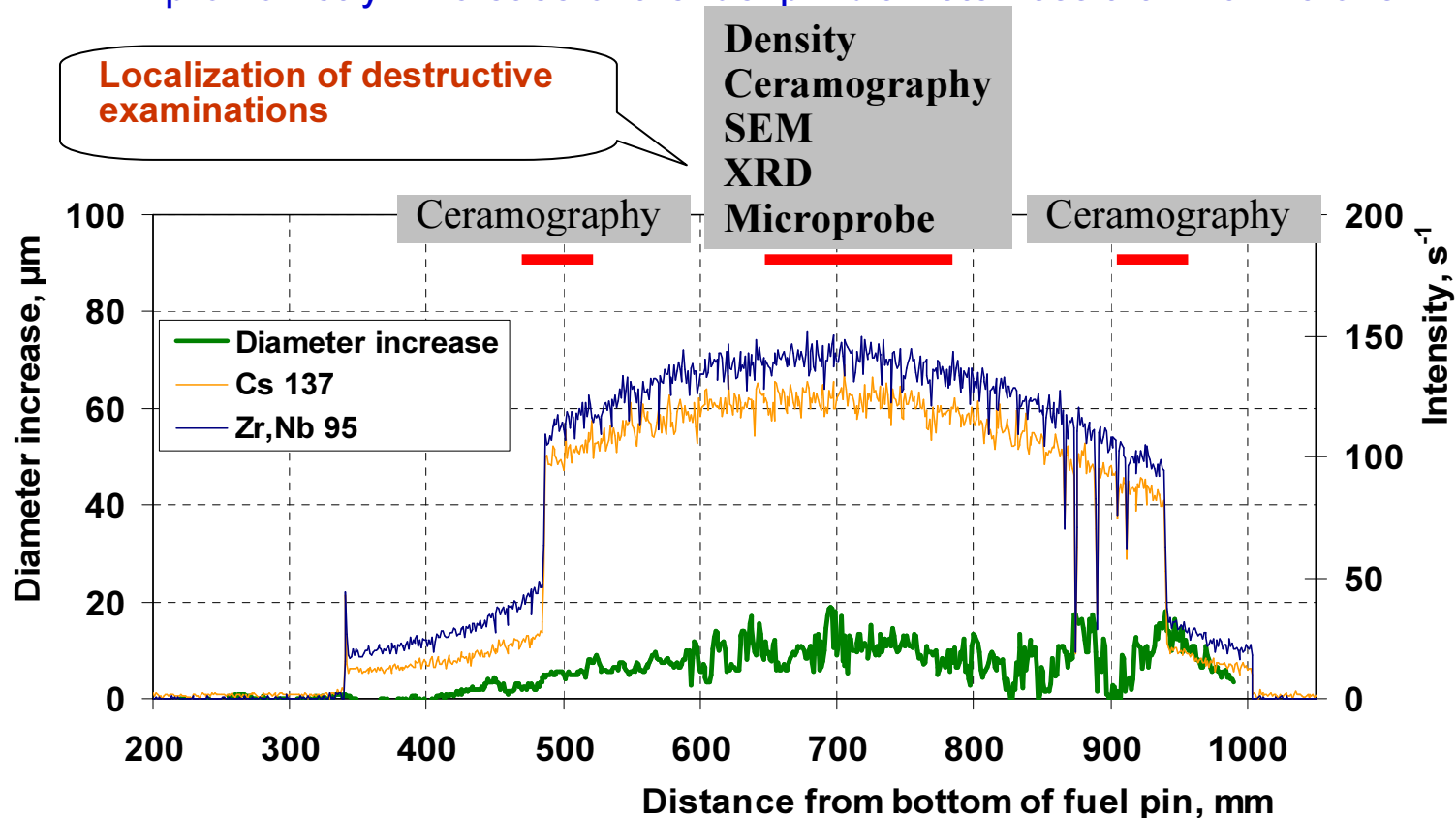
- ✓ visual control - there were no anomalies;
- ✓ leak-tightness control ( $^{85}\text{Kr}$ ) - leak-tight;
- ✓ gamma-scanning - no anomalies in the distribution of fission products;
- ✓ profilometry - increase of the fuel pin diameter less than 20 microns



## Similar results have been obtained for fuel pin with 40% $PuO_2$ +60% $MgO$ , burn up 19 h.a. %

### Non Destructive Testing:

- ✓ visual control - there were no anomalies;
- ✓ leak-tightness control ( $^{85}Kr$ ) - leak-tight;
- ✓ gamma-scanning - no anomalies in the distribution of fission products;
- ✓ profilometry - increase of the fuel pin diameter less than 20 microns



## ***GAS PHASE IN FUEL PINS ( $B \approx 19$ h.at. %)***

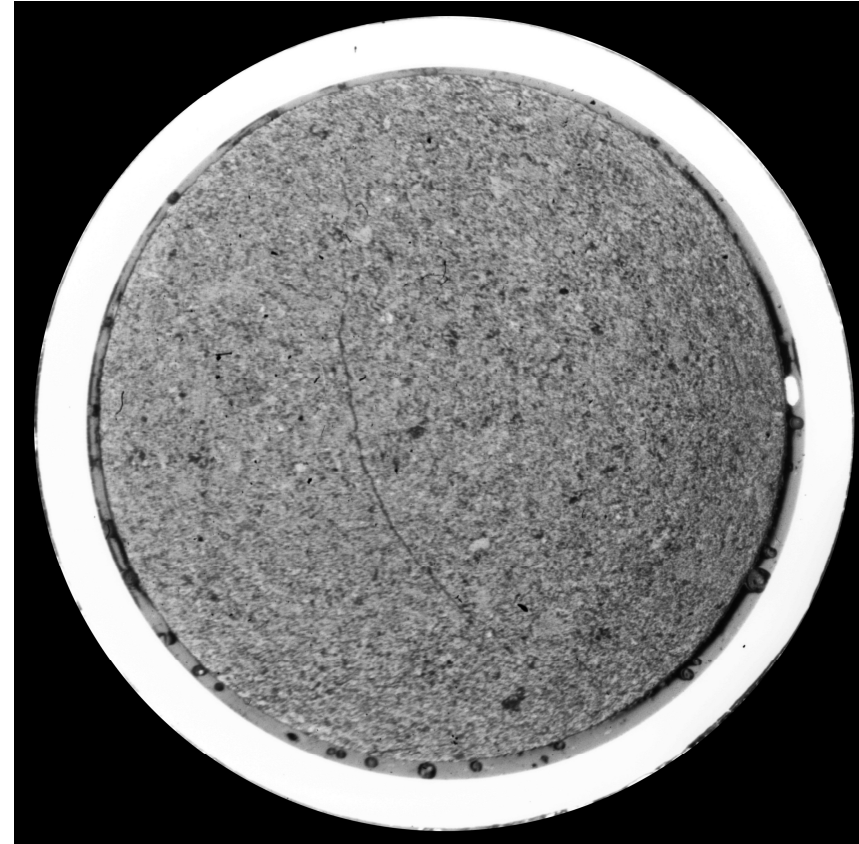
Fuel composition	Gas pressure in fuel pin, MPa (n.c.)	Gas volume, cm <sup>3</sup> (n.c.)	Gas content, vol.%			GFP volume, cm <sup>3</sup> (n. c.)	GFP release, %
			He	Kr	Xe		
<b>40%PuN +60%ZrN</b>	$0.09 \pm 0.02$	$11 \pm 2.6$	83	1	<b>12</b>	1.4	<b>1</b>
<b>40%PuO<sub>2</sub> +60%MgO</b>	$0.13 \pm 0.02$	$17 \pm 2.6$	62	2.4	<b>34</b>	6.1	<b>9</b>

**✓ Low level of gaseous fission products release**

## *Macrostructure of Irradiated composition*

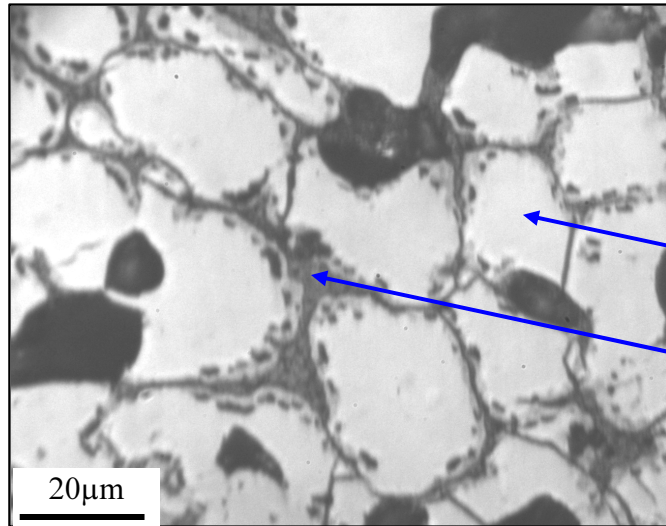


**40% PuN+60% ZrN**

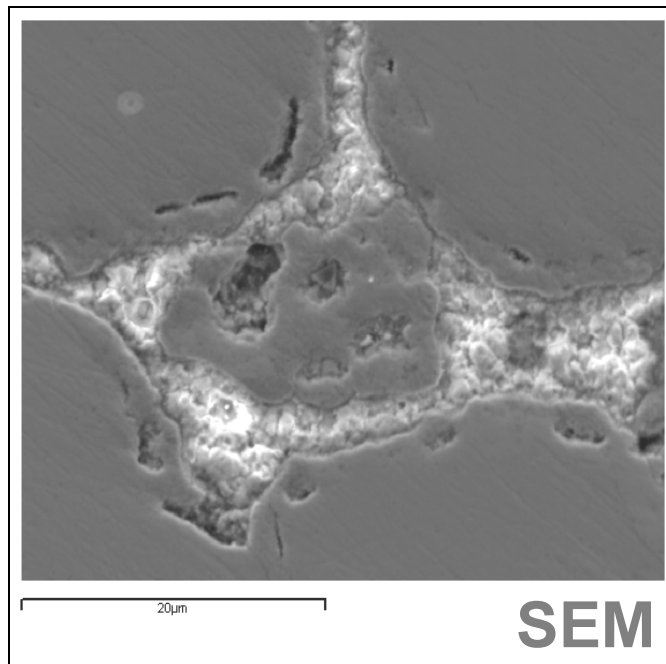


**40% PuO<sub>2</sub> + 60% MgO**

# DESTRUCTIVE EXAMINATIONS: 40%PuN+60%ZrN



Area of examination	Mass fraction, % (EPMA)							
	Pu	Zr	Nd	Xe	Cs	Mo	O	N
Light phase	25.1 ±2.4	61.1 ±3.1	0.55 ±0.05	0.8 ±0.3	0.64 ±0.13	0.56 ±0.01	1.2 ±0.4	10.9 ±3.8
Grey phase	74.4 ±1.4	6.9 ±1.4	0.9 ±0.2	0	0	1.14 ±0.08	13.5 ±1.7	0



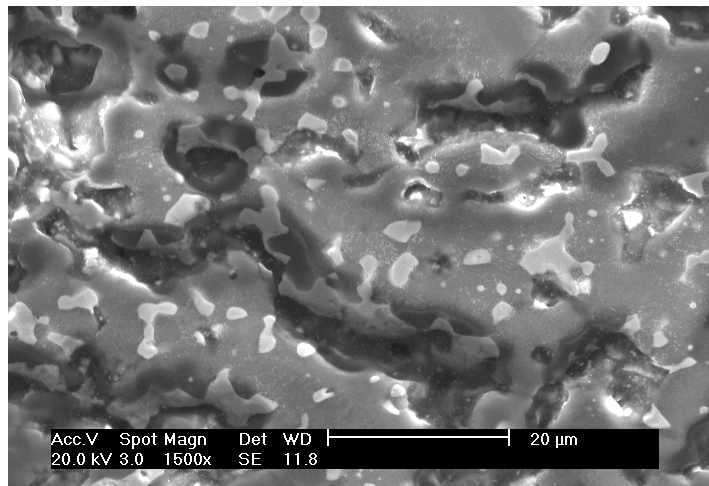
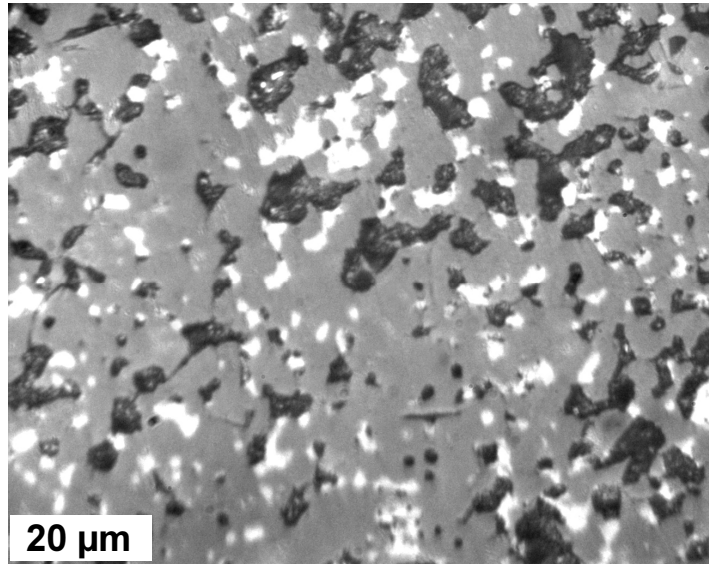
## Crystalline lattice parameter, nm

Unirradiated	0.476; 0.470 Pu(Zr)N; Zr(Pu)N
Burn-up 11.3 % at.	0.46432±0,00005 Zr(Pu)N
Burn-up 19.4 % at.	0.46484±0,00004 Zr(Pu)N
ZrN	0.4577
PuN	0.4905

- ✓ Two main phases – solid solutions based on zirconium and plutonium nitrides differing in their ratio and FP content



# DESTRUCTIVE EXAMINATIONS: 40%PuO<sub>2</sub>+60%MgO



SEM

Material	Crystalline lattice parameter, nm
Unirradiated fuel	0.5395 ±0.0001 (PuO <sub>2</sub> )
	0.5198 ±0.0001 (Pu <sub>2</sub> O <sub>3</sub> · 2PuO <sub>2</sub> )
	0.4211 ±0.0001 (MgO)
Burn-up 11.1 % at.	0.53885±0.00006; 0.42140±0.00001
Burn-up 19 % at.	0.53816±0.00004; 0.42178±0.00003
MgO	0.4211
PuO <sub>2</sub>	0.5396

✓ The irradiated fuel composition consisted of two phases - PuO<sub>2</sub> and MgO with the crystalline lattice parameters differing from those of pure chemical compounds;

✓ The EPMA showed the fission products in the plutonium-containing phase and in the magnesium-containing one.

## DENSITY AND SWELLING RATE OF FUEL COMPOSITIONS

<i>Fuel</i>	<i>Burnup, % h.at.</i>	<i>Density, g/cm<sup>3</sup></i>	<i>Average swelling rate, %/ 1% at.</i>
<b>40% PuN +60% ZrN</b>	<i>Unirradiated</i>	7.45 ± 0.05	<b>~ 0.1</b>
	11.1	7.46 ± 0.07	
	19	7.33 ± 0.04	
<b>40% PuO<sub>2</sub> + 60% MgO</b>	<i>Unirradiated</i>	4.46 ± 0.02	<b>~ 0.7</b>
	11.3	4.07 ± 0.04	
	19.4	3.92 ± 0.02	

✓ **Low Swelling rate of 40% PuN+60% ZrN**

✓ **Swelling rate of 40% PuO<sub>2</sub> + 60% MgO is about the same as of MOX fuel**

## **CONCLUSION**

**All the obtained results allow made a conclusion about the expediency of further investigation of**

***PuO<sub>2</sub>+MgO* and *PuN+ZrN***

**compositions up to higher burnup and at higher content of a fissile component.**