

**IRSN**

INSTITUT  
DE RADIOPROTECTION  
ET DE SÛRETÉ NUCLÉAIRE

*Faire avancer la sûreté nucléaire*

# Enhancing the Emergency Preparedness and Response Expertise with the Use of Advanced Response and Assistance Network Capabilities

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Emergency Response Department**



## RANET: Response and Assistance Network

RANET is an operational tool under the Assistance Convention which aims at facilitate:

- ▶ The provision of requested international assistance
- ▶ The harmonization of emergency assistance capabilities and,
- ▶ The relevant exchange of information and feedback of experience

IAEA Action plan on Nuclear Safety which gives a new mandate to the Agency regarding the provision of information to MS and the general public during a nuclear emergency on its potential consequences, including available information and prognosis of possible scenarios...

## RANET: Response and Assistance Network

### 8 functional areas including:

- ▶ Radiological Assessment and Advice
  - ▶ Atmospheric Dispersion
  - ▶ Dose prediction...
- ▶ Nuclear Installation Assessment and Advice
  - ▶ Nuclear Power Reactor Accident Analysis
  - ▶ Spent Fuel Storage Assessment and Advice...

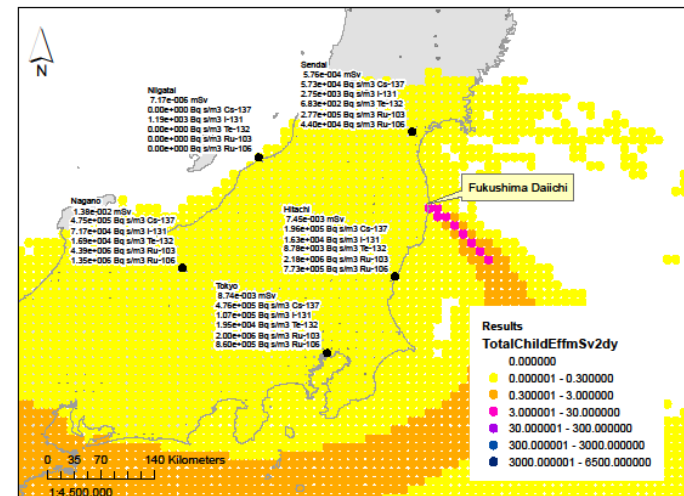
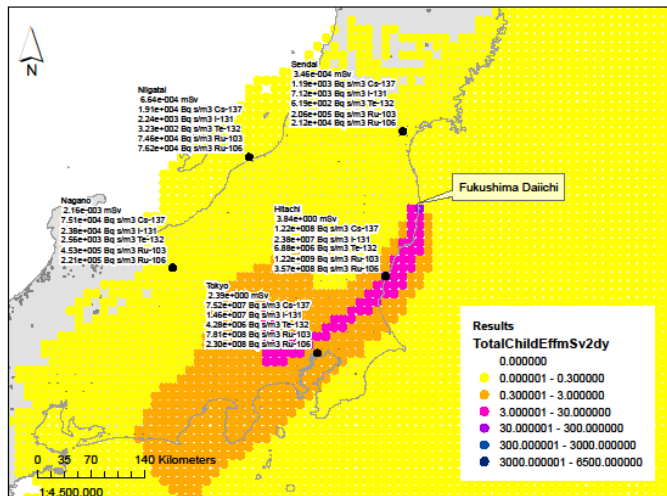
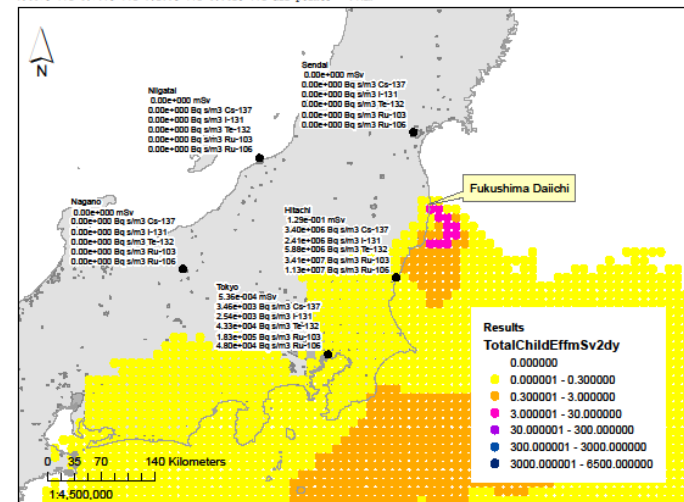
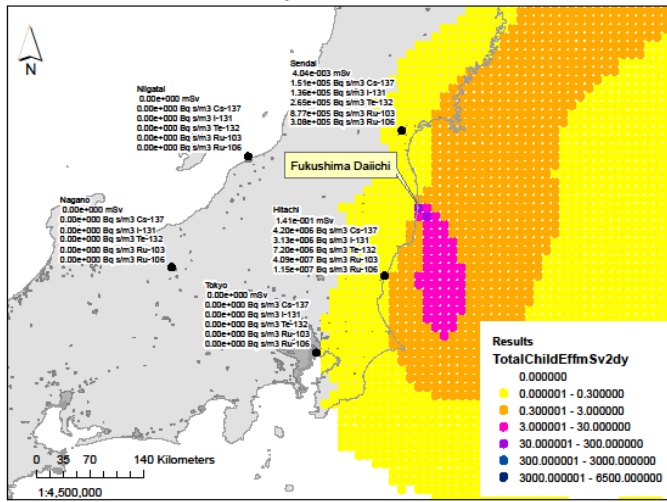
The IEC (Incident and Emergency Centre) aims at developing an Assessment and Prognosis process to fulfill the new mandate

## An accident somewhere is an accident everywhere

During the response to the Fukushima emergency (all) MS with expertise capabilities where providing analysis and anticipation scenarios to their strategic/decision level.

- ▶ Using pre-calculated source terms (unit, scenario based, real-world accident)
- ▶ Using live calculated source terms from analysed data from TEPCO public data available (diagnostic and prognostic)
- ▶ Using their National Meteorological Services for weather forecasts at different scale
- ▶ Using their national response system (models) for atmospheric transport and for dose assessments

## UK (PHE, RIMNET & Met. Off.)



## Russia (IBRAE)


 ТЕХНИЧЕСКИЙ КРИЗИСНЫЙ ЦЕНТР  
 ИНСТИТУТ ПРОБЛЕМ БЕЗОПАСНОГО РАЗВИТИЯ АТОМНОЙ ЭНЕРГЕТИКИ

Оперативному дежурному СКЦ Росатома  
 Срочно для Генерального директора Госкорпорации «Росатом»  
 Кириенко С.В.  
 Копию – заместителю генерального директора Госкорпорации  
 «Росатом» Локшину А.М.  
 Копию – директору Департамента коммуникаций Госкорпорации  
 «Росатом» Новикову С.Г.

12/03/11  
 СПРАВКА № 9  
 19:30

ДЛЯ ИНФОРМАЦИИ! СРОЧНО!

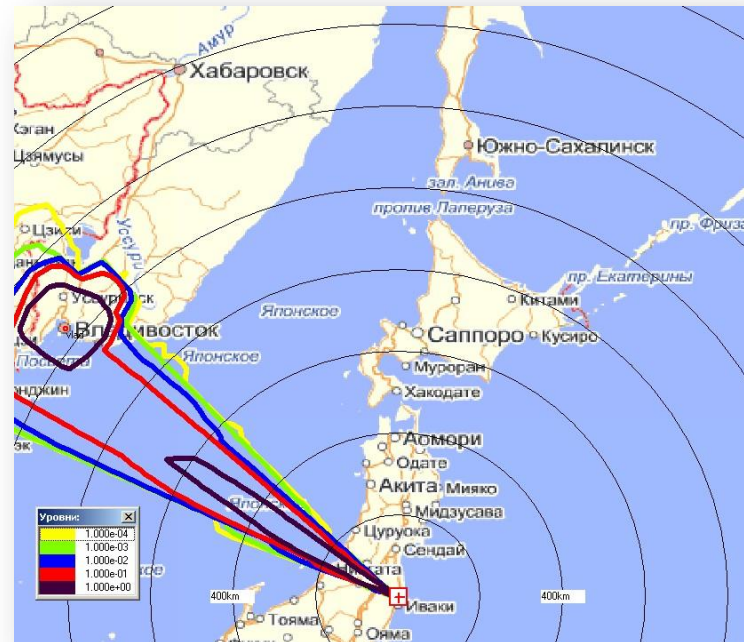
Расчетный анализ консервативного по сценарию протекания аварии на АЭС с  
 максимальными значениями (мадореалистичными) возможного выброса с  
 неблагоприятными метеоусловиями с направлением переноса облака в  
 территории Российской Федерации (расчетная точка – г.Владивосток)  
 выполненной в ИБРАЭ РАН, показывает, что прогнозируемые дозы  
 облучения даже в случае выпадения осадков над территорией г.Владивостока  
 значительно меньше значений, представляющих сколь-либо значимый ущерб  
 здоровью и не требуют каких-либо мер вмешательства.

Максимальная аварийная доза при абсолютно нереалистичных  
 предположениях возможной величины выброса будет составлять 10 мЗв за  
 год. Мощность дозы при этом кратковременно может возрастать до 20-30  
 фоновых значений, т.е. 2-3 мЗв/ч (200 – 300 мкР/ч).

С помощью модуля БОНУС 2.1 в составе кода СОКРАТ проведены  
 предварительные оценки накопления нуклидов во время нормальной  
 эксплуатации АЭС Fukushima Daiichi (блок № 1).

В таблице 1 приведены параметры активной зоны реактора при которых  
 проводились расчеты.

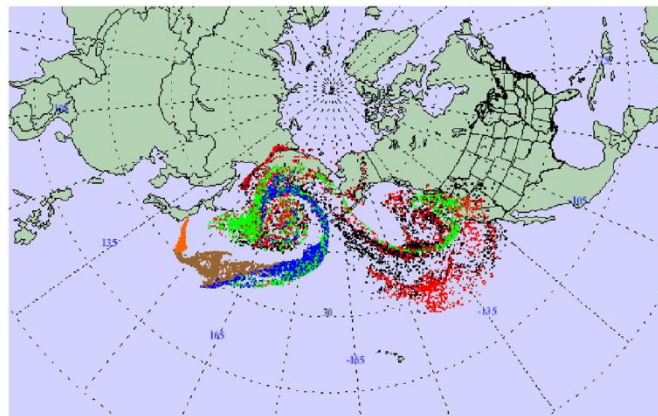
**The worst (unlikely) weather conditions have been  
 chosen for calculation:**  
**Wind speed – 10 m/s,**  
**Wind direction - 115 degrees,**  
**Stability category – E,**  
**Local precipitations in the Vladivostok area  
 at the rate of 10 mm/h.**



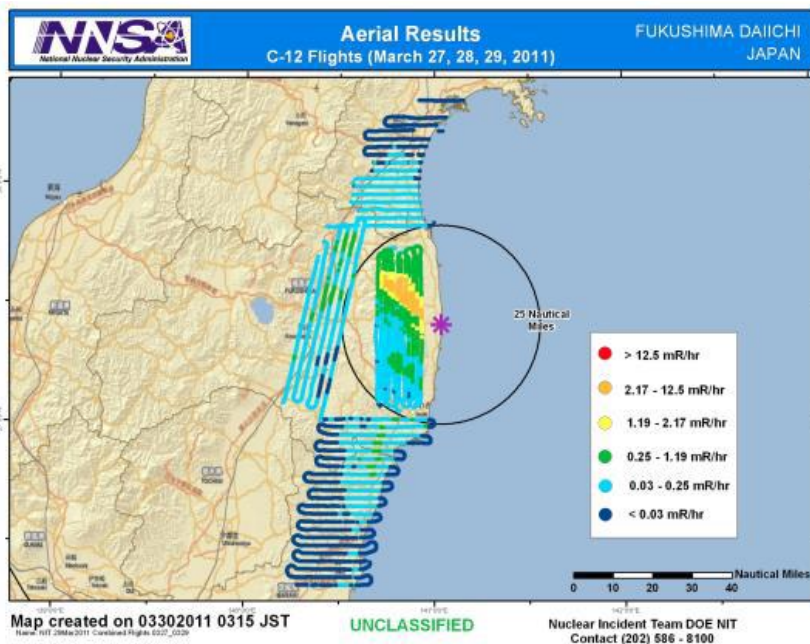
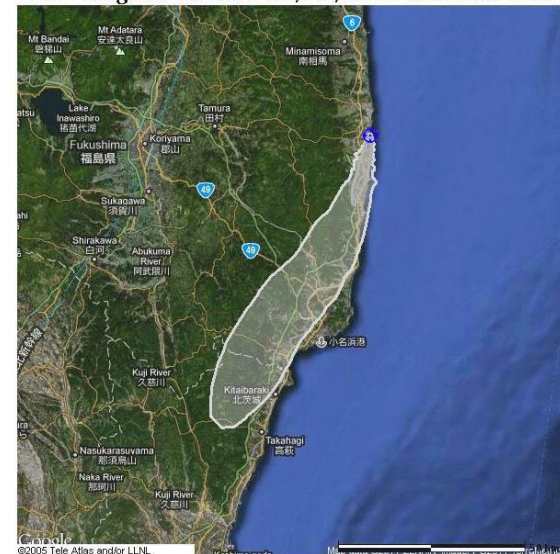
**Total annual effective dose  
 (children of 1-2 years) within 10 mSv**



## USA (US-NRC & US-DOE: NARAC)



1hr-Avg Air Conc at 03/22/2011 03:00:00 UTC



## France (IRSN)



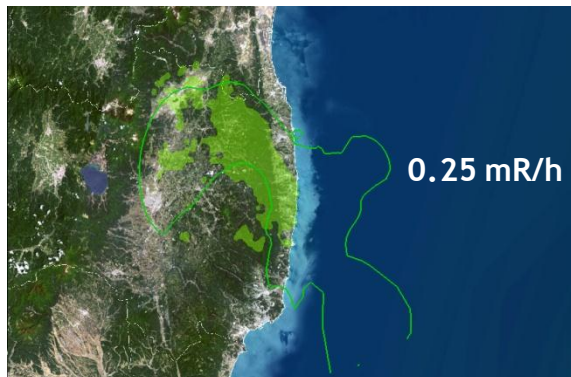
22 mars 2011

L'IRSN publie une évaluation de la radioactivité rejetée par la centrale de Fukushima Daiichi (Fukushima I) jusqu'au 22 mars 2011

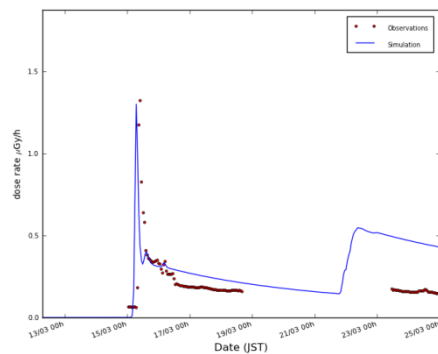
Annexe

Composition détaillée du rejet utilisé pour estimer les niveaux de contamination de l'air

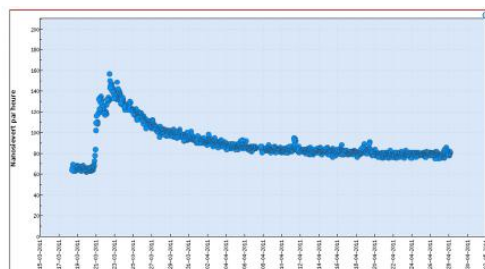
Isotope	Cumul. (Bq)	Isotope	Cumul. (Bq)	Isotope	Cumul. (Bq)	Isotope	Cumul. (Bq)
Kr-85	2 E+16	I-131	9 E+16	Cs-134	1 E+16	Rb-88	5 E+13
Kr-85m	1 E+14	I-132	7 E+16	Cs-136	6 E+15	Rb-89	3 E+02
Kr-87	7 E+11	I-133	2 E+16	Cs-137	1 E+16	Te-133m	4 E+10
Kr-88	5 E+13	I-134	4 E+11	Cs-138	3 E+09	Te-134	6 E+09
Xe-133	2 E+18	I-135	2 E+15	Cs-134m	1 E+12	Sb-130	1 E+15
Xe-133m	2 E+16	I-129	2 E+09			Sb-125	6 E+14
Xe-135	2 E+16	I-132m	2 E+10			Sb-127	4 E+15
Xe-138	9 E+01	I-128	4 E+04			Sb-128	1 E+10
Kr-83m	1 E+13					Te-127	5 E+15
Xe-131m	2 E+16					Sb-128m	1 E+13
Xe-135m	6 E+14					Sb-129	4 E+13
						Te-129m	7 E+15
						Sb-131	8 E+05
						Te-125m	1 E+14
						Te-132	6 E+16
						Te-127m	1 E+15
						Te-131	5 E+14
						Te-131m	2 E+15
						Te-133	7 E+09
						Br-83	2 E+12
						Br-84	7 E+07



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Most (if not all) of these expertise were done without any exchange

## Mainly because objectives were different

- ▶ Protection of national residents living in Japan (Tokyo)
- ▶ Planning of ground (or airborne) missions (rescues, monitoring...)
- ▶ Planning and response to plume arrival (monitoring) on MS territory
- ▶ Domestic public communication including evaluation of health impact on MS territory
- ▶ None (almost) where devoted to look at the protection of local population around the Fukushima plant

**Data needed to provide expertise (for domestic use and as a RANET asset) shall be organized and distributed through a centralized system**

The Fukushima accident showed that at some point, the accident state must provide raw data of the on-going accident (installation, environment) to let expertise assets throughout the world use them for their own analysis (needed by MS to take strategic decisions)

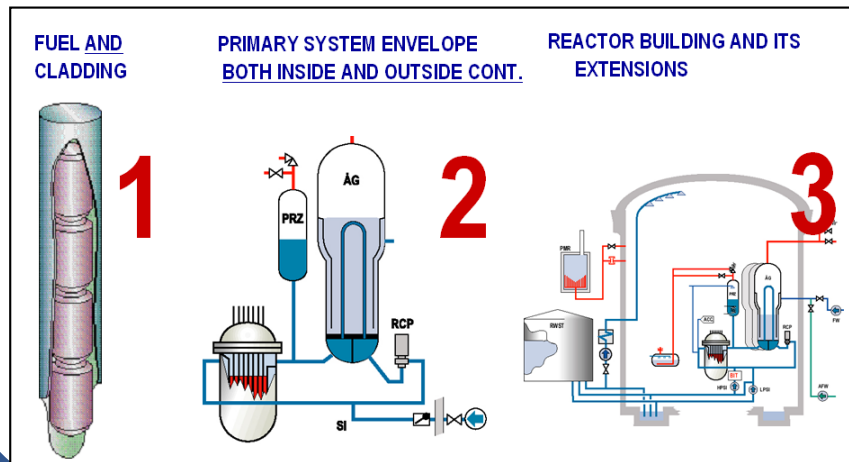
- ▶ **Avoid too much transfer is of paramount importance to minimize effort from the accident state and to insure consistency of distributed data**
- ▶ Accident related data
  - ▶ Dynamic data from the impacted nuclear installation
  - ▶ Environmental monitoring data on and off-site
- ▶ Meteorological data
  - ▶ Forecasts at different temporal resolution and spatial scales
  - ▶ Observation and Nowcast products

## What for?

Enhancing the understanding of the situation is done through the use of a structured methodology providing a regular diagnostic of the state of the accident installation.

Anticipation on possible scenarios and development of the accident shall also be done with the help of a structured methodology

The IRSN methodology (3D/3P) structure the work of expertise resources, facilitate the dialogue and the exchange of information between expertise teams and **does not require numerous data to work !**



COUNTRY	NPP SITE	NPP UNIT	COORDINATES LONG/LAT	REACTOR TYPE	DATE	LOCAL TIME	GMT TIME
FRANCE	GRAVELINES	1	0 / 50	Generic PWR			
STATUS at 15 h 54							
DIAGNOSIS							
Barriers Status	Safety functions status	Systems used for the monitoring of functions	Forecast availability of systems	Forecast status of safety functions	Forecast Status of Barriers		
<b>Fuel Matrix and Claddings</b>	<ul style="list-style-type: none"> <li>comfortable</li> <li>low</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>control rods</li> <li>boration: SI, chemical &amp; vol. ctrl sys, makeup, FBA</li> </ul>	<ul style="list-style-type: none"> <li>comfortable</li> <li>low</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>control rods</li> <li>boration: SI, chemical &amp; vol. ctrl sys, makeup, FBA</li> </ul>	<b>Fuel Matrix and Claddings</b>		
<b>No cladding Failure</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>Cladding Failure</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>Core Melt</b>	<input type="checkbox"/>	<ul style="list-style-type: none"> <li>satisfactory</li> <li>degraded</li> <li>dewatering</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>satisfactory</li> <li>degraded</li> <li>dewatering</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>SI, chemical &amp; vol. ctrl</li> <li>water reserves (RWST, makeup)</li> </ul>	<ul style="list-style-type: none"> <li>SI, chemical &amp; vol. ctrl</li> <li>water reserves (RWST, makeup)</li> </ul>		
<b>Primary System Integral</b>	<input type="checkbox"/>	<ul style="list-style-type: none"> <li>SG</li> <li>break</li> <li>SI</li> </ul>	<ul style="list-style-type: none"> <li>SG</li> <li>break</li> <li>SI</li> </ul>	<ul style="list-style-type: none"> <li>SG</li> <li>break</li> <li>SI</li> </ul>	<b>Primary System Integral</b>		
<b>Doubtful</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>Primary break</b>	<input type="checkbox"/>	<ul style="list-style-type: none"> <li>controlled or not</li> <li>adequate or not</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>controlled or not</li> <li>adequate or not</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>feed and bleed</li> <li>Decay heat and Removal system</li> <li>Component Cooling sys.</li> <li>Essential Service Water system</li> </ul>	<ul style="list-style-type: none"> <li>Decay heat and Removal system</li> <li>Component Cooling sys.</li> <li>Essential Service Water system</li> </ul>		
<b>Reactor Building and its Extension Normal Leakage</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>Doubtful</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>Uncollected leakage</b>	<input type="checkbox"/>	<ul style="list-style-type: none"> <li>safe</li> <li>not guaranteed</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>safe</li> <li>not guaranteed</li> <li>doubtful</li> </ul>	<ul style="list-style-type: none"> <li>containment isolation phases 1 and 2</li> <li>containment spray</li> <li>system in service</li> </ul>	<ul style="list-style-type: none"> <li>containment isolation phases 1 and 2</li> <li>containment spray</li> <li>system in service</li> </ul>		
<b>Collected leakage</b>	<input type="checkbox"/>				<input type="checkbox"/>		
<b>RB depressurization</b>	<input type="checkbox"/>				<input type="checkbox"/>		

## Example of simple data exchange between the operator and IRSN Technical Emergency Centre

- ▶ Need to establish at the preparedness phase the data to be exchanged
- ▶ Automatic data transmission is preferable
  - ▶ Allow to run expertise in parallel
  - ▶ Does not require extra resources from the operator

Division Production Nucléaire	ORGANISATION DE CRISE PLAN D'URGENCE INTERNE CHAPITRE A 2.2			INDICE 01	PAGE 07/07		
C.N.P.E. de PENLY IRSN Centre Technique de l'Opérateur	POSTE DE COMMANDEMENT LOCAL (PCL)			D 5039 - ODCIA.2.2			
Folio 2/2							
<b>MESSAGE QUART D'HEURE</b>							
ETAT DE LA TRANCHE : <input checked="" type="checkbox"/> RP - <input checked="" type="checkbox"/> AN/GV (cocher)							
Site : PENLY	Tranche : 3	Date : 23/06/05	Heure du relevé : 9h00				
Rédacteur PCL 2	Nom : Schmitt Visa	Vérificateur PCL 1	Nom : ... Visa				
Destinataires : PCC, PCM, ETC-N, CTC-IRSN, SEPTEN, FRAMATOME							
* Entourer la mention utile ** Unité à préciser suivant l'échelle de l'enregistreur							
T : Tendance du paramètre : → ↗ ↘ ↙							
PARAMETRES	GV 1	GV 2	GV 3	GV 4	T CAPTEURS, ...		
Niveau GV Gamme Large	%	100	100	100	100	ARE 108 à 408 MN	Pos, Pos
Isolément GV côté eau		oui	oui	oui	oui	TPL	Pos, Pos
Isolément GV côté vapeur		oui	oui	oui	oui	TPL	Pos, Pos
Débit ASG	m <sup>3</sup> /h	23	20	0	30	ASG 102 à 402 MD (Gamme étroite) ASG 101 à 401 MD (Gamme large)	Pos, Pos
Niveau bache ASG	m <sup>3</sup>	1250				ASG 01 ou 02 MN	Pos
Débit APG	t/h	0				APG 051 MD	P10
Pression GV	bar	12	16	18	12	WP 105 à 405 MP	Pos
Activité purges GV	Bq/m <sup>3</sup>	Energie/Kwh			H2O/Kwh	KRT 11, 12, 13, 14 MA	P22
Activité vapeur GV	Bq/m <sup>3</sup>	1			1	KRT 15, 16, 17, 18 MA	P21
Pompes primaires en service		oui	oui	oui	oui	Ampèremètre	Pos
Température branche froide	°C	?	?	?	?	RCP 104 à 404 MT	Pos
Température branche chaude	°C	145	?	?	145	RCP 100 à 400 MT	Pos
Niveau cuve	%	81				Ebulliomètre	Pos
Température sortie coeur max (TRIC)	°C	130				Ebulliomètre	Pos
Marge à la saturation coeur (ΔTSAT)	°C	454				Ebulliomètre	Pos
Marge à la saturation couvercle (ΔTSAN)	°C	41				Ebulliomètre	Pos
Pression primaire	bar	9.5				Ebulliomètre	Pos
Flux niveau intermédiaire	A	10 - 11				RPN 13, 23, 33, ou 43 MA	Pos
Débit ISMP (voie A et voie B)	m <sup>3</sup> /h	0	0	0	390	RIS 47 et 48 MD	T04
Débit ISBP (voie A et voie B)	m <sup>3</sup> /h	0	0	0	0	RIS 27 et 28 MD	T04
Débit de charge	m <sup>3</sup> /h	0				RCV 28 ou 48 MD	P04
Débit de décharge	m <sup>3</sup> /h	0				RCV 09 MD	P04
Niveau pressuriseur	%	0				RCP 12, 13, 17 ou 18 MN	Pos
Niveau bache PTR	m <sup>3</sup>	2000				PTR 12, 13, 14 ou 15 MN	T04
Débit aspersion enceinte	m <sup>3</sup> /h	Voie A : 0	Voie B : 1600			EAS 01 MD / EAS 02 MD	T05
Pression enceinte	bar rel.	0.14				RIS 003 ou 004 MP	T05
Température enceinte	°C	95				RIS 005 MT ou 006 MT	T05
Débit de dose Gamma enceinte	mGy/h	< 10				KRT 40 ou 43 MA	T06
Seuils niveau pulsard RIS		voie A : Max1 (0.52) / voie B : Max1 (0.52)				RIS 11 et 12 AA	T03
Débit à la cheminée	m <sup>3</sup> /h	160000				DVN 160 ou 165 MD	P01
Activité β gaz normale cheminée	Bq/m <sup>3</sup>	10 <sup>4</sup>				KRT 02 ou 84 MA	P01
Activité β gaz accident cheminée	MBq/m <sup>3</sup>	< 10 <sup>4</sup>				KRT 05 ou 89 MA	P01
Procédure incidentelle ou accidentelle en cours (ou séquence) : ECP4 PAF conduite dure arrêt ISMP							
Actions en cours, événements et actions survenus (par exemple : ouverture des soupapes du pressuriseur, isolement de l'enceinte, niveau bas PTR, ...): Refroidissement à 56°C/A							
Événements et actions prévus : Suite à la coupure des batteries voie A → perte de l'aléa voie A et de l'information au SdC (enregistreur, ...)							

Most of the advanced centers which provide nationally expertise on radiological consequences during response are not meteorological services, they are able to run operational models which evaluate the atmospheric dispersion processes and the various doses for the public and consequences for the environment.

Their arrangements with national meteorological services make available weather forecasts to them to be in a position to do Radiological Assessment

Providing assistance as a RANET asset require to have access to “best” weather forecast products possible in the impacted area but also world-wide.

**WMO shall organize RMSCs to provide such meteorological products to IAEA which will then broadcast them to RANET assets in charge of Radiological Assessment and Advice.**



Contribution from different organizations to the Assessment and Prognosis process will enhance the global capabilities to be in a position to understand a developing situation and resulting possible consequences

The provision of several Assessment and Advice to the IAEA/IEC during a response will enable the IEC to combine results to produce concerted messages with the Accident State.

In doing so, the IEC will stay at a strategic level providing to MS a global overview of the situation and possible developments, enabling requesting MS to plan and to implement adequate responses to the situation

The Assessment and Prognosis process is achievable if potential Accident State commit to provide the necessary technical data (dynamic measurements in the installation) and if WMO and IAEA put in place a mechanism for RANET assets to get access to meteorological forecast products

- ▶ Extension of existing expertise methodologies to all type reactors
- ▶ Improve reference codes for Severe Accident (source term model), including the improvement of Iodine behaviour models
- ▶ Provision of operational ensemble forecasts
- ▶ Improve wind direction forecast at small scale and ‘Nowcast’ products (including rain radar)
- ▶ Improve multi-scale atmospheric dispersion models (including deposition) and chemistry models for iodine in the atmosphere
- ▶ Improve of Data Assimilation methodology on environmental measurements (retrieve of source terms)