

Safety Research Activities on Severe Accident Management in S/NRA/R after Fukushima Daiichi Nuclear Power Plant Accident

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1. New regulation on NPP

- New regulatory requirements for NPPs came into force in July 2013
- Legally requested measures to prevent and to mitigate severe accidents (SAs)
- Regulation based on the state-of-the-art information
 - Develop new regulatory standards and apply them to existing nuclear facilities (back fitting rule)
- 40-years operation limit for NPPs
 - Legally define the limit to 40years
 - NRA can permit a less-than-20-year extension



1. New regulation on NPP

<Previous Regulatory Requirements> <New Regulatory Requirements >



*SSCs: Structure, Systems and Components



Ref; Enforcement of the New Regulatory Requirements for Commercial Nuclear Power Reactors http://www.nsr.go.jp/english/e_news/data/13/0912.pdf

1. New regulation on NPP New Requirements on SAM

- Prepare multi-layered protection for "prevention of core damage", "maintaining containment integrity" and "suppression of radioactive materials dispersion"......
 - Over pressure/Over temperature ; Less than designed value or PCPL (primary containment pressure limit)
 - ✓ Hydrogen treatment; H₂ conc. in PCV < 13% (without steam)</p>
- Enhance reliability with permanent systems/equipment and prepare mobile equipment.
 - Redundancy, diversity, independence, dispersion
- Enhance protective measures in spent fuel pool.
 - Water level measurement, alternative water supply,...
- Improve command communication and instrumentation.
 - Reinforced seismic-resistance of on-site emergency response center, improved reliability/durability of communication system,...



Ref.: Outline of New Regulatory Requirements For Light Water Nuclear Power Plants (Severe Accident Measures), http://www.nsr.go.jp/english/data/new_regulatory_requirements 3.pdf

2. Safety Research Activities in S/NRA/R Objectives of Safety Research Programs

- Research on development of evaluation criteria for regulatory standards and requirements
 - Technical evaluation criteria for regulatory standards and guidelines on safety review
 - Development and maintenance of analysis code etc.
- Research on acquisition of technical knowledge necessary for decision making of nuclear safety regulatory actions
 - Technical data to increase the accuracy of verification for safety review and inspection etc.
- Research on the maintenance and establishment of technical basis
 - To maintain technical basis to properly implement regulatory activities in the future.

RA Japan

2. Safety Research Activities in S/NRA/R Focused Safety Research Topics

- Special emphasis on external / internal hazards leading to large scale common cause failure:
 - Extreme natural phenomena:
 - Hazard curves of earthquake/tsunami, fragilities of SSCs , ...
 - PRA methods and models: External/internal fire and floods, multihazards, multi-units, application of level 3 PRA

Research on SAs:

- **Code development** for SA progression / source terms, ...
- Experiments on scrubbing, seawater injection, ...

Research on Fukushima Daiichi NPP Accident:

Management of wastes/contaminated water, ...

Other areas:

Decommissioning/waste Disposal, fuel cycle facilities, ...



3. Overview of Research on SAs Objective and Outcome for Research on SA

Major Objectives

- To develop and improve analysis code, and to promote experiments for the decrease of the uncertainties of SA phenomena.
- To understand SA phenomena or progression that may occur under potential accident conditions and to investigate new measures to mitigate SAs based on the up-to-date scientific and technical knowledge.
- To contribute to international collaboration by participating in international Joint research projects to study the common subjects.

Outcomes

- Proposal for development of the regulatory standard or guideline related to SA for NPPs.
- Confirmation of the effectiveness of proposed licensing actions for preventing SA.

3. Overview of Research on SAs SA Subjects based on Fukushima Daiichi NPP Accident



KA Japan

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3. Overview of Research on SAs Safety Research Plan by S/NRA/R

Near term experimental subjects

- Pool scrubbing test ; Study on scrubbing under saturated water conditions taking into account flashing accompanied by venting.
- Seawater injection test ; Study on the influence on fuel/debris cooling by seawater/boric acid injection
- Inorganic iodine behaviours test ; Study on pH dependence of inorganic lodine behaviors
- Mid & long term experimental subjects
 - Core melting and debris formation test; Study on evaluation method of ex-vessel debris coolability by cavity injection strategy



- Development of SA code based on the results of experiments
 (According to the progress of experiments)
 - SA analysis code for plant system behaviour
 - Mechanistic SA analysis code for core melting and debris formation

4. Example of Research Activity Pool Scrubbing Test (1/2)

Background

- Rapid decompression due to venting might cause saturated boiling over a wide range in S/P and degrade scrubbing effect on reducing the release of radioactive materials in the case that S/P temperature keeps high for a long period.
- This process is still controversial subject, however, clarifying this phenomenon is crucial in determining the source term.

Objective

 To clarify aerosol physics and two-phase fluid dynamics taking into account flashing accompanied by venting for improving AM measures.



Flashing accompanied by venting₀



4. Example of Research Activity Pool Scrubbing Test (2/2)

Small Scale Test

- Perform systematic parametric studies to separate underlying mechanisms.
- High resolution instrumentation including visualization



□ Large Scale Test

- Large scale tank with the pool depth equivalent to that of BWR plants.
- Obtaining data that can be easily scaled up to the actual plant
 Di va conditions.



2 m

30 m³

Diameter:

Volume:



Max. Pressure > 1MPa Max. Temperature > 200 °C

Items	2012	2013	2014	2015	2016	2017	2018~
Scrubbing Test							
Small Scale Test							
Large Scale Test							
Modeling, SA Code Improvement							



4. Example of Research Activity Seawater Injection Test (1/2)

Background

- Seawater is regarded as one of alternative water source in the accident management (AM) in Japan.
- It is important to understand how long and how much seawater injection can continue to reactor pressure vessels (BWRs or PWRs) without losing the cooling effect.

Objective

NRA Japan

 To identify the salt and boric acid crystallization/precipitation characteristics and its influence on fuel/debris cooling such as flow blockage for improving AM measures.



4. Example of Research Activity Seawater Injection Test (2/2)

Debris Bed Test Fuel Bundle Test Observe influences on the heat Seawater/boric acid Monitor formation solution tank removal of debris bed in the process of Separator lower head by several mock up precipitation by X-Spacer crusts. ray CT Downcomer Online sampling needles Seawater Observe influences Solution Tank Salt **Coolant Spray** precipitation on the heat removal area Pool by mockup of BWRs imulated Debris and Crust fuel bundle Spacer Overflow Pre-heater Heat Concluction Plate Tank Seawater/boric acid Bottom 6x6 mockup boiling loop bundle Heater

Thermo-Couples for Heat Transfer Surface Temperature

Items	2012	2013	2014	2015	2016	2017	2018~
Bundle Test							
Debris bed Test							
Development of Heat Transfer Models							



5. Summary

- After Fukushima Daiichi NPP Accident, severe accident research program is placed on one of important research activities in NRA
- S/NRA/R established R&D plan to decrease the uncertainties of SA phenomena and has conducted a part of it in near term experiments,
 - Near term experimental subjects are in progress on schedule.
 - Analysis models will be developed and improved based on these experimental data contributing to the reduction of uncertainly of SA analysis result.
 - The detail of results on experiments and analysis will be addressed in a timely manner.
- S/NRA/R participates in the several international cooperative projects continuously to resolve the remaining issues on SA phenomena and to develop and improve the analytical codes.



Summary of Safety Research in NRA

Appendix

Categories Safety Research Topics/Subjects Safety analysis and code development Thermal-hydraulic phenomena during accidents at LWRs Sever accidents 1. Nuclear Reactor Regulatory criteria for fuel Aging technical evaluation and extension of operation limit Management of water chemistry/quality Management of radioactive wastes including liquid wastes 2. Fukushima Daiichi Transportation of damaged spent fuel Evaluation of design basis earthquake and assessment of seismic ground motion/land slides Evaluation of design basis tsunami and probabilistic tsunami hazard 3. Internal or External Structural integrity assessment for earthquake and tsunami Hazards Research on volcano eruption for new regulatory requirements Evaluation techniques for fire protection Risk evaluation for internal and external hazards leading to common cause failure Evaluation techniques for interim storage and transportation of radioactive materials 4. Nuclear Fuel Cycle Assessment for aging management for fuel reprocessing facilities Methods for verification of clearance 5. Backend Evaluation techniques for safety review of underground waste disposal 6. Emergency Response There's no specific topic at the moment. 7. Physical Protection It needs to maintain technical capabilities. 8. Radiation Protection and Monitoring Human and organizational factors 9. Cross-cutting Issues Criticality assessment of spent fuel Maintaining technical competence/expertise

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