AP1000: The PWR Revisited



IAEA

International Conference on Opportunities and Challenges

for Water Cooled Reactors in the 21st Century

aolo Gaio

estinghouse Electric Company



ouse

Background

- Late '80: USA Utilities under direction of EPRI and endorsed by NRC : Advanced Light Water Reactor Utility Requirements Document (URD) with policy and design requirements for next generation
- URD addresses evolutionary and passive LWR
- Passive has much higher expectations (ex. maintain safe shutdown for 72 hrs. after design base event w/o operator action vs. 30' for evolutionary)
- Passive is also simpler, smaller and much improved
- In Europe similar document European Utility Requirements (EUR)

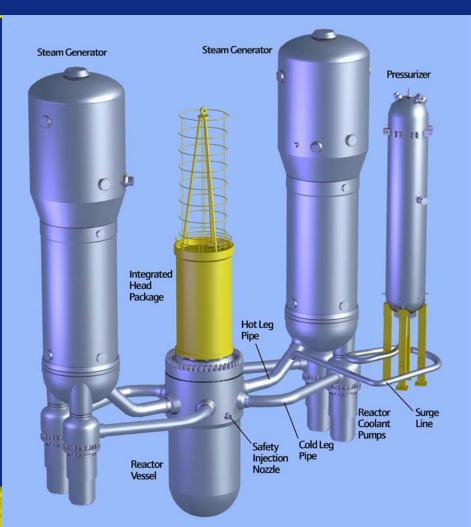


AP 1000: The Reactor Coolant System



Familiar but Improved Reactor Coolant System

- Two loop plant: two cold and one hot leg
- Four Reactor coolant pumps mounted in steam generator lower head - No RCP shaft seals
- Larger pressurizer (2100 ft -50% larger than operating plants) eliminate PORV
- Top-mounted, fixed in-core detectors
- Primary pipes forged in one piece reduces welds 50%, supports 80%
- Ring-forged reactor vessel (no longitudinal welds)
- All-welded core shroud (not bolted)
- Fuel, Internals, Reactor Vessel
 - Similar to Doel 4, Tihange 3, S. Texas
 - Improved materials 60 yr life
- Steam Generators
 - Similar to large Westinghouse SGs in operation

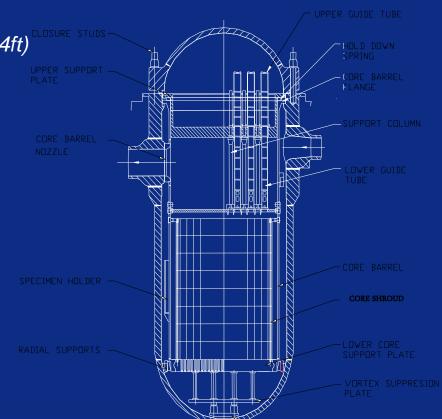




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AP1000 Reactor Vessel

- Westinghouse 3-loop Reactor
 - 3.99m (159in) ID, *157 fuel assemblies*
 - Longer vessel to accommodate 4.27m (14ft) fuel
 - Ring forged construction
 - No welds in core region
 - Improved materials permit
 60- yr design life
 - W-CE-type core shroud
 - Replaces radial reflector
 - All-welded design
 - Top-mounted in-core I&C
 - Fixed position, online readout
 - No penetrations in bottom vessel

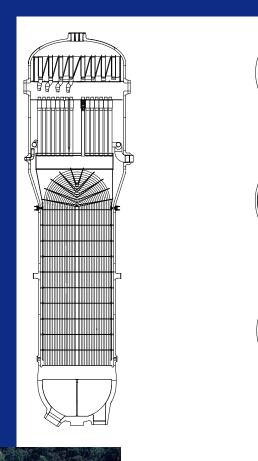


SECONDARY CORE SUPPORT



AP1000 Steam Generator

- Based on proven Westinghouse designs
 - AP1000 design based on ANO RSG
 - Design Features
 - Inconel 690 TT tubes
 - Stainless steel support plates
 - Improved access
 - Excellent operating experience
 - Over 1,200 SG years of operation
 - Less than 0.1% total tubes plugged

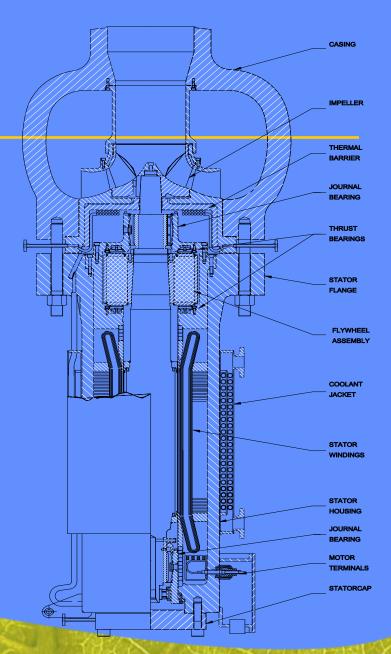


AP1000 High Efficiency Reactor Coolant Pump

- Canned Motor Pump
 - No shaft seal leakage or support system
 - No seal (LOCA benefit)
 - No oil cooling system
 - Reduced maintenance
 - Excellent operating experience
 - High inertia bearing

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- Pump design will be verified through extensive testing





Comparison of Selected Parameters

PARAMETER	Doel 4 / Tihange 3	AP1000
Net Electric Output, MWe(2.5"HgA)	985	1117
Reactor Power, MWt	2988	3400
Hot Leg Temperature, °F	626	610
Number of Fuel Assemblies	157	157
Type of Fuel Assembly	17x17	17x17
Active Fuel Length, ft	14	14
Linear Heat Rating, kW/ft	5.02	5.71
R/V I.D., inches	157	157
Vessel Thermal Design Flow, gpm	295,500	299,880
Steam Generator Surface Area, ft ²	68,000	125,000
Reactor Coolant Pump Flow, gpm	103,400	78,750
Pressurizer Volume, ft ³	1400	2100



AP 1000: The Passive Safety System



Passive Safety – What's it all about?

- Passive Safety Systems utilizes naturally occurring physical phenomena such as natural circulation of air, water and steam
- Gravity and gas pressure drive the flow of cooling water
- **Natural heat transfer** occurs through conduction, convection and evaporation
- There are no safety related pumps and motor-operated valves
- There is no safety related ventilation system
- Reactor safety functions are achieved without using any safety related
 AC power rely on "stored" energy
- A few battery powered valves 20 in total, most "fail safe" align the passive safety systems upon actuation signals ("one time" alignment)



Passive Safety – What's it all about? (Cont.)

- Typical PWR Safety Systems and Safety-Related Support System exist as simplified, non-safety systems housed in non-safety related structures (defense in depth):
 - Diesels
 - Chilled Water
 - Component Cooling
 - Spent Fuel Cooling
 - Diesel Support Systems (Fuel Oil, HVAC)

- Instrument Air
- Hot Water (heating)
- Essential Service Water
- Residual Heat Removal
- Startup Feedwater

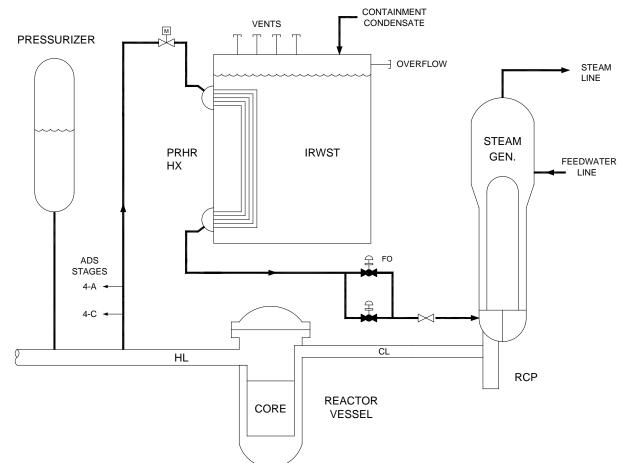
(replaces Auxiliary Feedwater)

- The Non-Safety, Active Systems are credited in the PRA, but are <u>not</u> required for reactor safety and are <u>not</u> required to achieve NRC required CDF of 1x10⁻⁴
- Severe Accident Scenarios are mitigated by In-Vessel Retention of the melted fuel. (The core is retained in a cooled reactor vessel)



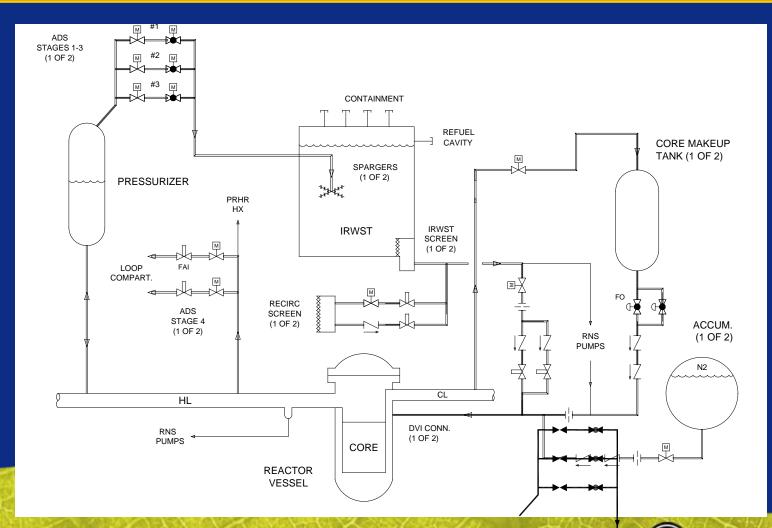
Passive Decay Heat Removal

- PRHR (Passive Residual Heat Removal) normally isolated by two AOVs, fail open
 - Opening 1 AOV actuates RCS cooling via natural circulation
 - AOVs actuated by PMS and by DAS
- IRWST absorbs heat
 - Takes ~ 2 hours to heat up to saturated
 - Steaming is condensed by PCS and returned to IRWST by gutter





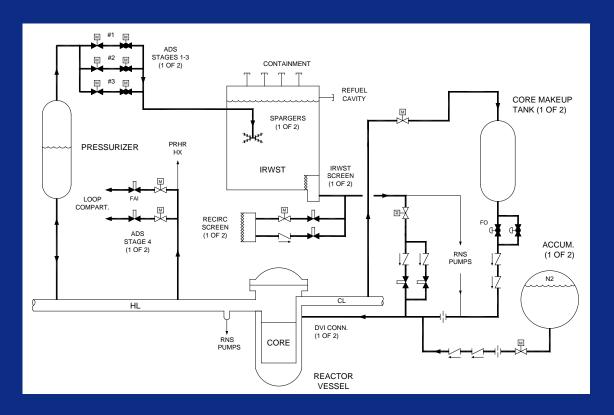
Passive Safety Injection





Passive Safety Injection

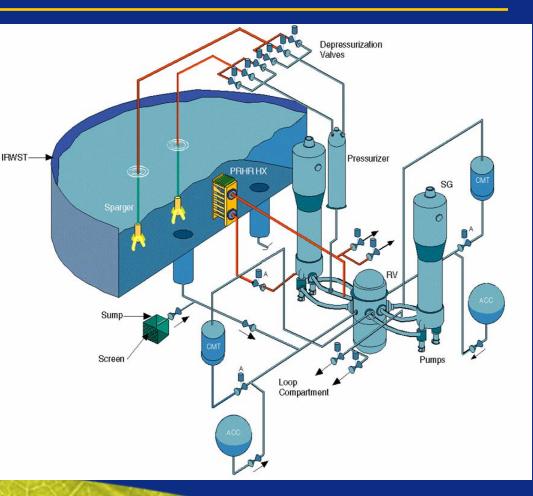
- Uses one time valve alignment
 - Accumulator uses check valves
 - CMT uses fail open AOV
 - ADS uses MOV for 1/2/3 and Squibs for 4
 - IRWST uses
 Squibs and check valves





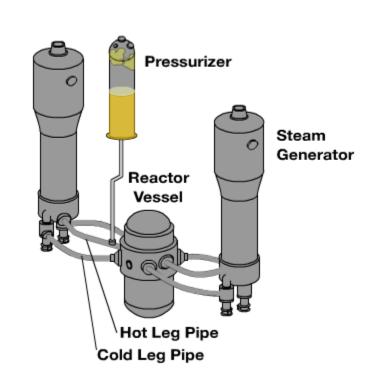
Simplicity in Design and Safety

- No operator action for 72 hours
- AP1000 safety doe not rely on AC power
 - Passive decay heat removal
 - Passive safety injection
 - Passive containment cooling
- Uses passive safety systems
 - Proven by extensive testing and analysis
 - Extensively reviewed by USNRC
 - No safety pumps
 - No ac power required
 - One time valve alignment Most are fail safe
- Provides improved margins
 - Transient DNBR margin > 15%
 - No core uncovery for SBLOCA
 - Breaks up and including a Direct Vessel Injection (DVI) line (8") break
 - No operator actions required for SG Tube Rupture





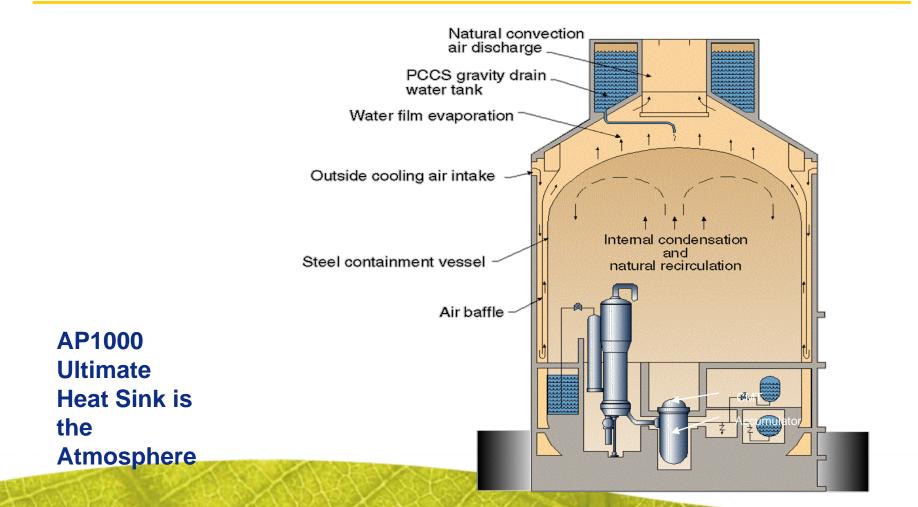
Passive Core Cooling System Operation During a Small-Break LOCA







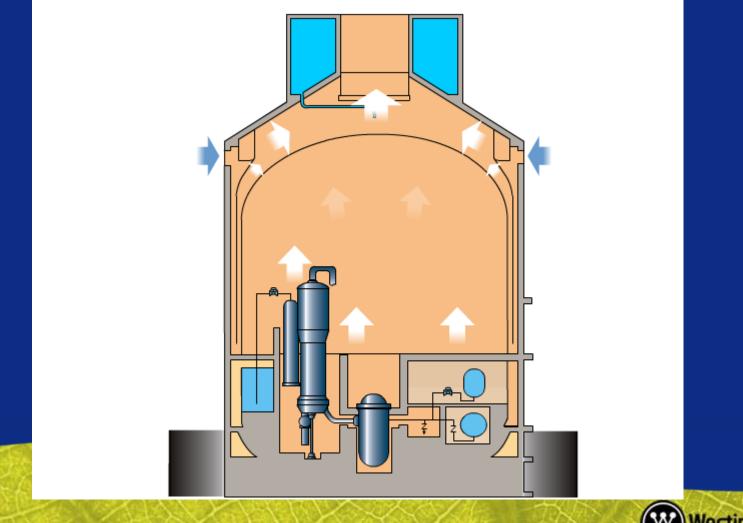
Passive Containment Cooling System



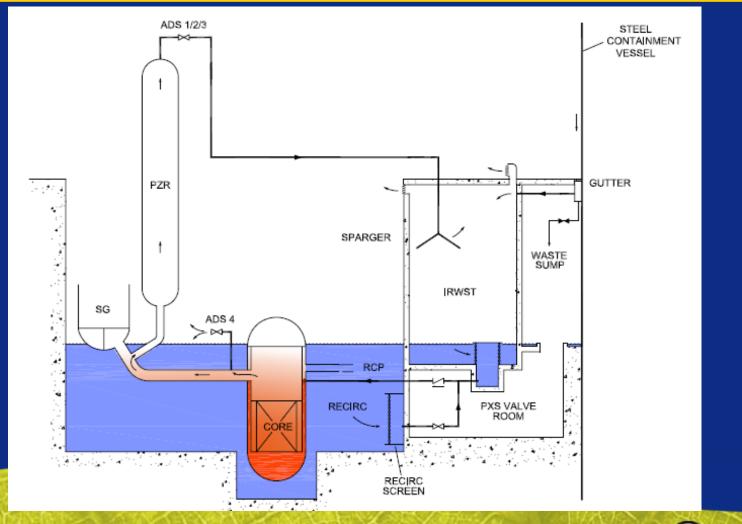
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Passive Containment Cooling Operation During a LOCA



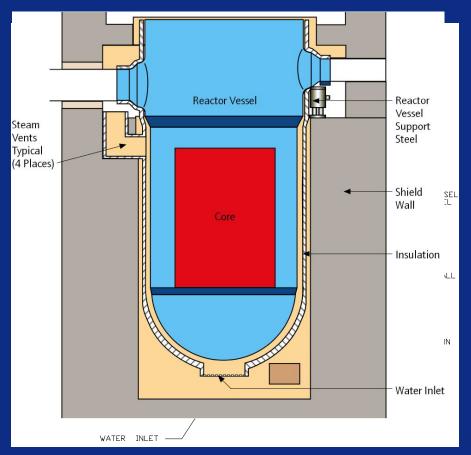
LOCA Long-Term Cooling





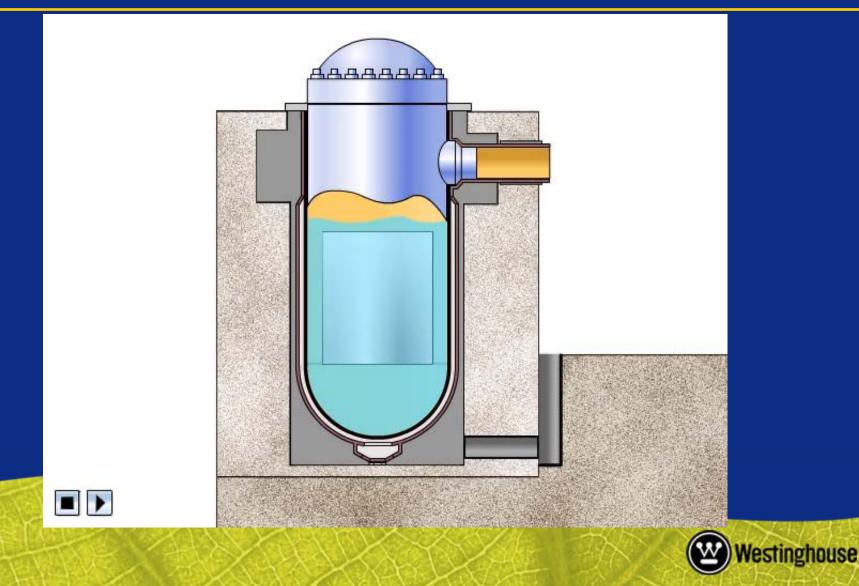
Simple Severe Accident In-Vessel Retention

- AP1000 designed to retain core debris <u>within the reactor</u> <u>vessel</u>
- Provides Reliable Means of Cooling Damaged Core
 - Tests and analyses reviewed by U.S. NRC
- In PRA Core Damage Sequences
 - Cooling flow driven by natural circulation
 - Water source: In containment refueling water storage tank
 - Cooling water flow path in vessel/ insulation annulus
 - Core heat transferred through RV wall
 - Water in containment removes heat from RV
 - ADS valves keep RCS pres low
 - Passive containment cooling transfers heat out of containment
 - Core debris retained inside reactor vessel
- Large release frequency: 5.9 x 10-8 per reactor year; URD requires < 10-6





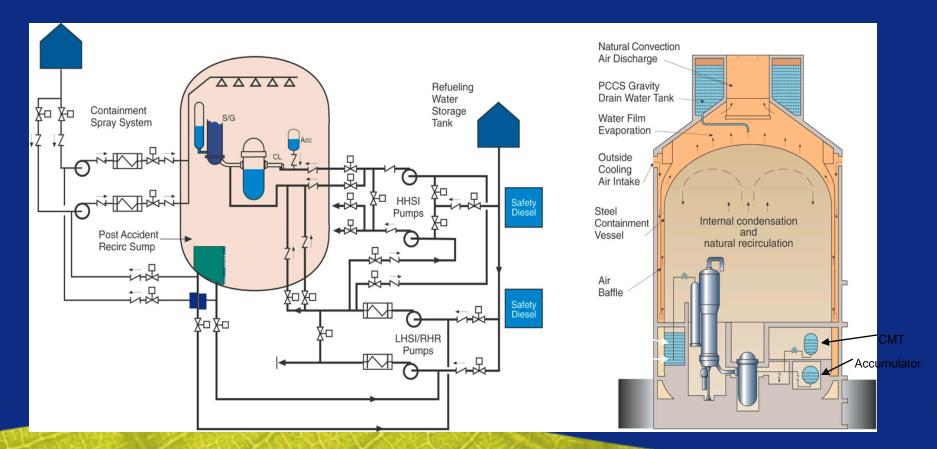
In-vessel Retention for Severe Accident Management



Simplification of Safety Systems Dramatically Reduces Building Volumes

Standard PWR

AP1000





All Advanced Reactors Can Achieve Low Core Damage Frequency

- Evolutionary plants achieve goals by **adding** redundant safety features
 - 4 Train Safety Injection
 - 4 Train Decay Heat Removal
 - 4 Train Containment Cooling
 - 4 Train Residual Heat Removal
 - 4 Train Diesel Generators

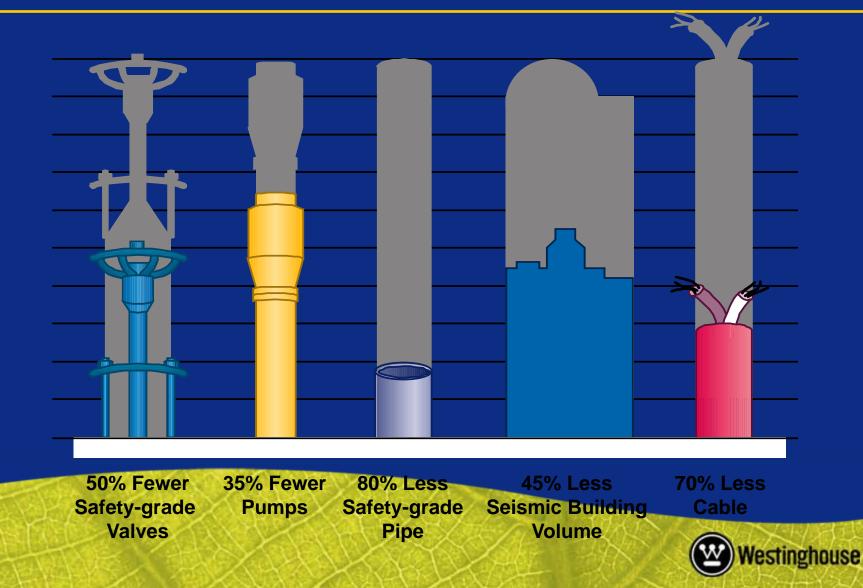


- Passive Plants achieve goals by **reducing** active safety features
 - No safety related pumps
 - No safety related fans
 - No safety diesels/no safety AC power
 - Small number of valves actuate passive systems
 - Natural forces provide plant safety

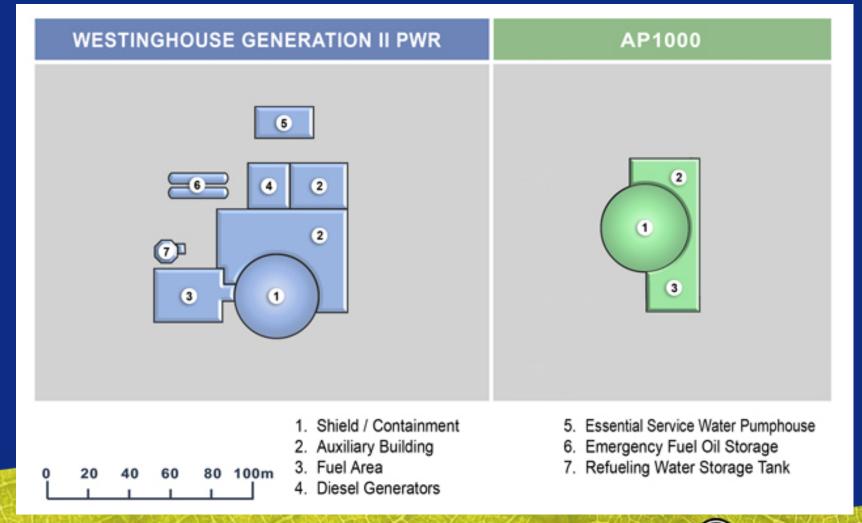
REDUCED CAPITAL AND O&M COST



AP1000 Passive Safety System Design Improves Economics and Construction Schedule



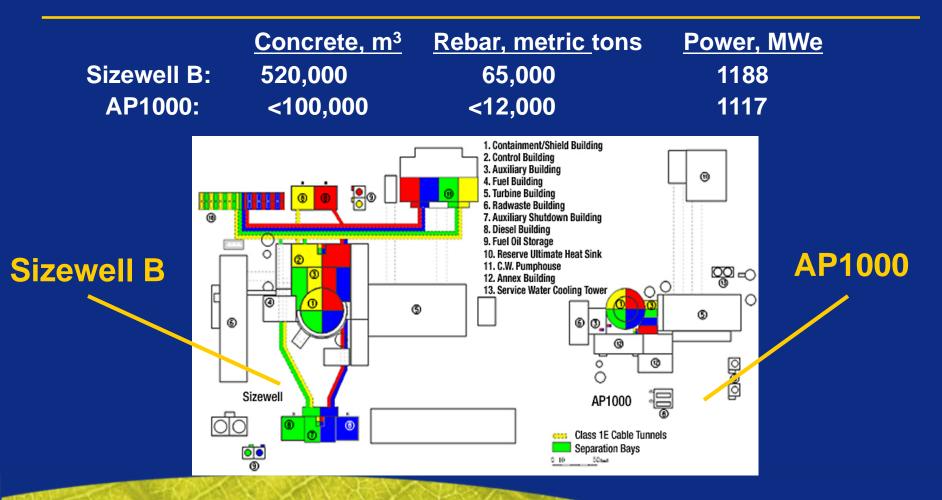
Comparison of Seismic Category I Buildings





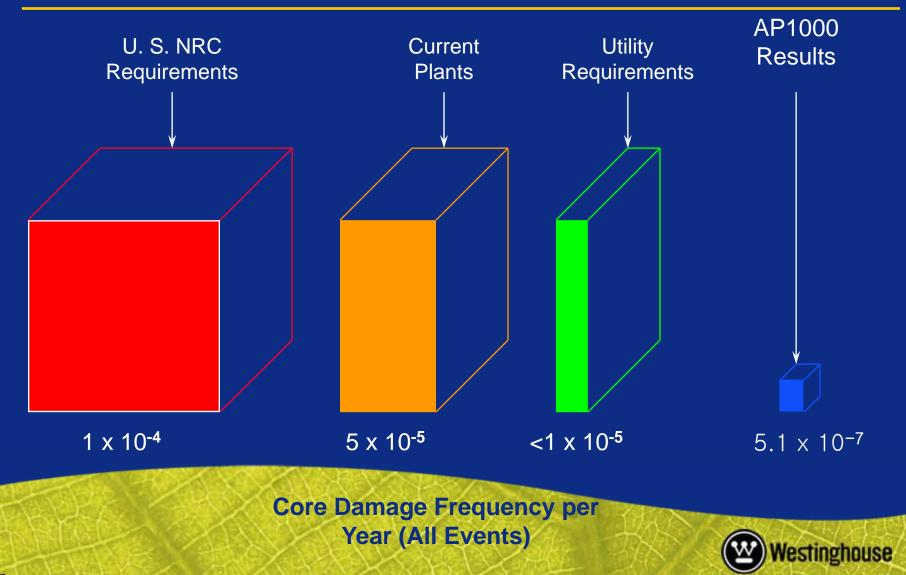
AP1000 Construction Simplification

Think: 1) more power/m³ of concrete, 2) less to decommission





AP1000 Provides Safety and Investment Protection



AP 1000: The Most Tested Reactor



AP 1000 Mature design

- AP 1000 is the most tested reactor (USA, Japan, Italy)
- Licensing
 - NRC *(total staff4,000 people)* performed **independent and confirmatory plant tests** (ROSA Japan, Oregon State University)
 - NRC spent 110 man-year effort over 6 years (W spent 1,300 man-years...)
 - NRC asked 7,400 written questions
 - Finally NRC issued Design Certification in Dec. '99
- Meets UDR (US utility requirements)

Meets EUR (European utility requirements)

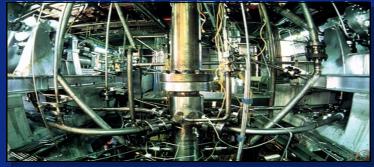


Thorough Testing of AP1000 Passive Systems

- Definitely the Most Tested Reactor -



Oregon State ¼ Scale, Long Tern Integral Systems Test



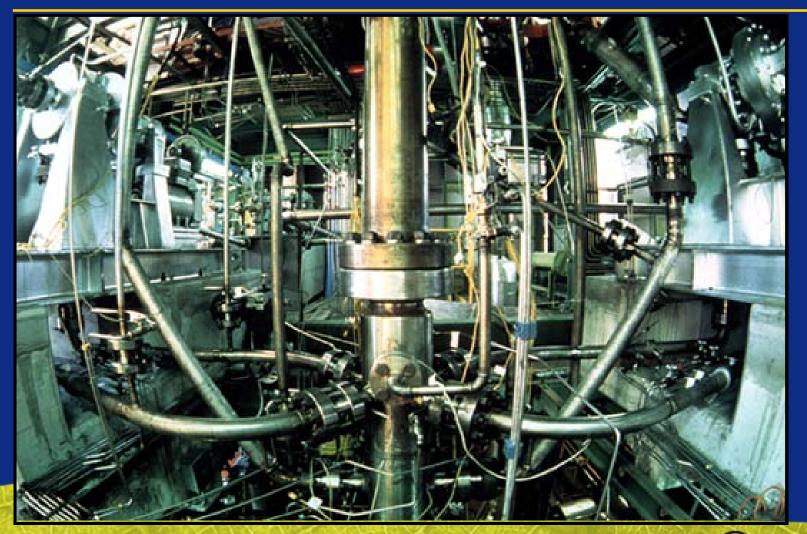
Full Height, Full Pressure, Integral Systems Test (SPES)



Large-Scale Containment Heat Transfer PCS Test

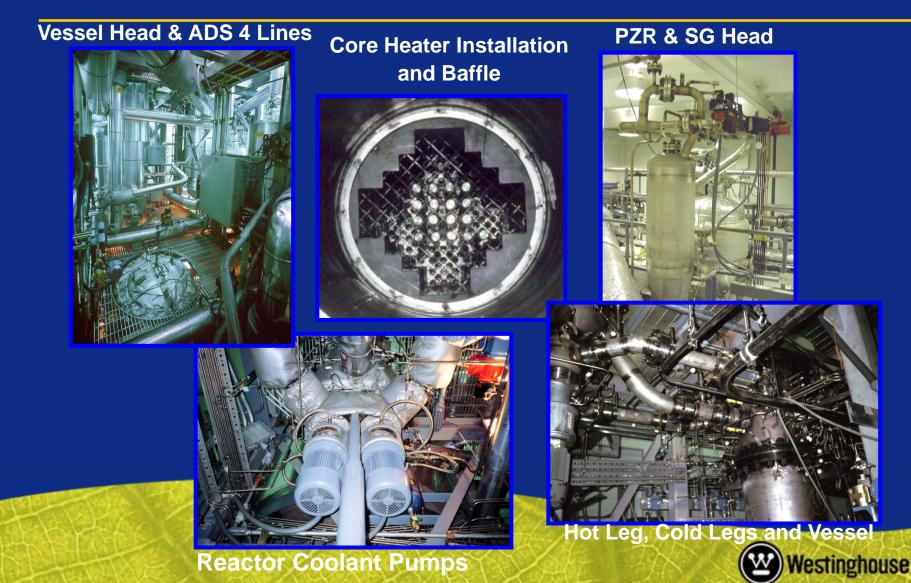


Full Height, Full Pressure, Integral Systems Test (SPES-2 - Italy)





APEX Facility (USA) Description



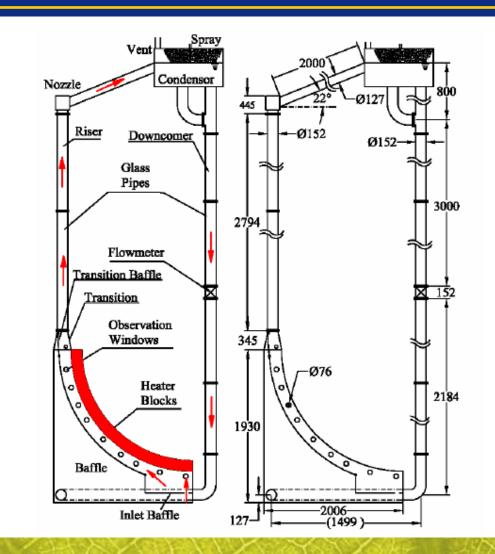
Large-Scale Heat Transfer PCS Test -USA





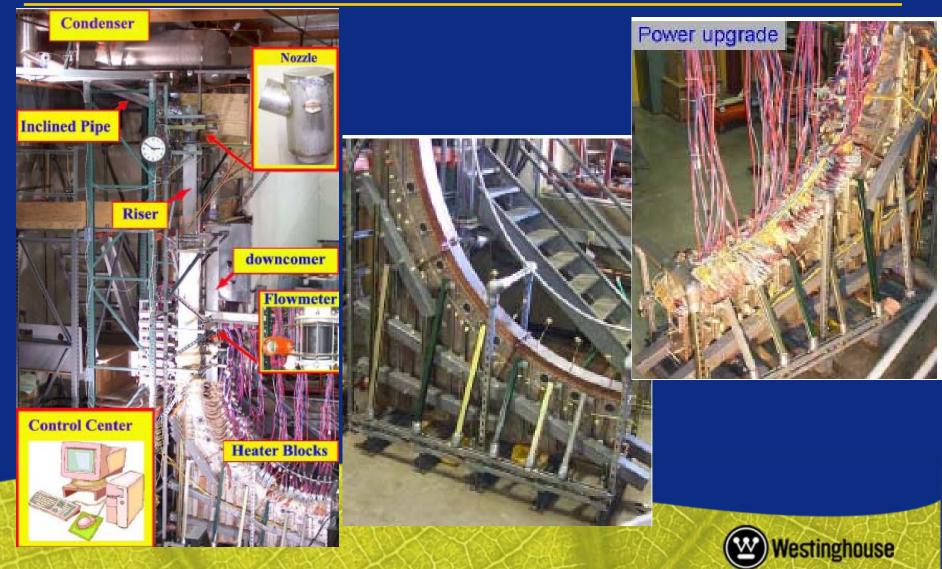
ULPU-2400 Config. V Full Scale Slice Testing

Confirms IVR Heat Transfer from Outside Reactor Vessel





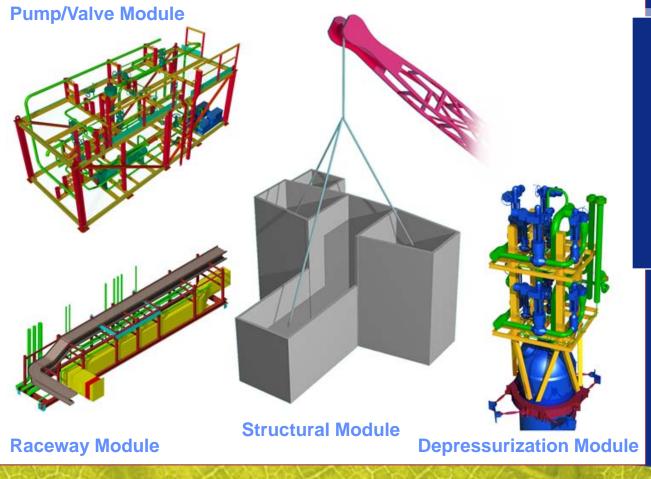
ULPU-2400 Config. V Full Scale Slice Testing Confirms IVR Heat Transfer from Outside Reactor Vessel



AP 1000: The Modular Construction



Modules Designed into AP1000 from the Beginning



<u>Module Type</u>	<u>Number</u>
Structural	122
Piping	154
Mechanical Equipment	55
Electrical Equipment	11
TOTAL	342

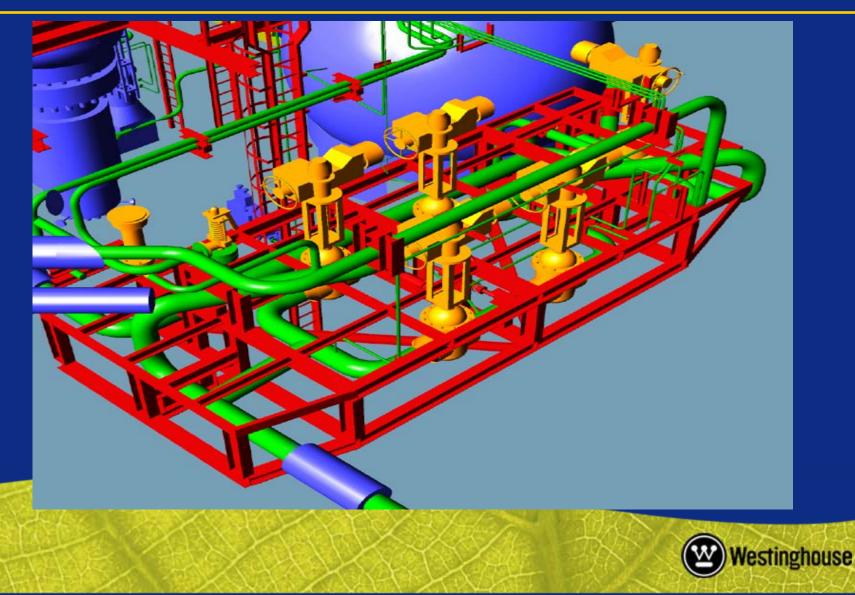


Modularization – Impact on Construction

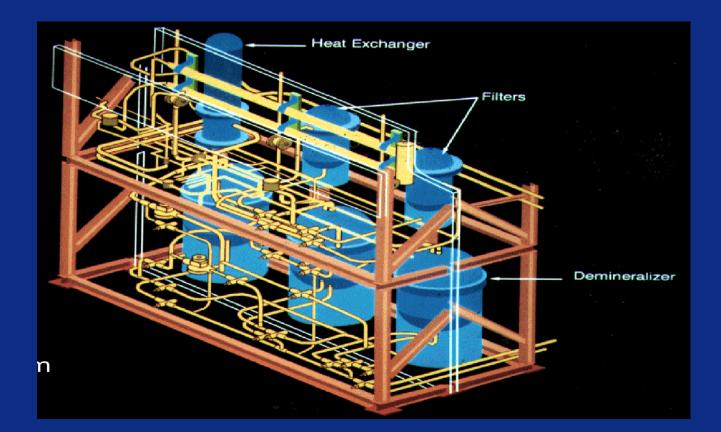
- Reduced calendar time, site labor and overall risk:
 - Module fabrication performed parallel with civil / structural work
 - Optimizes and levels manloading for mechanical and electrical work
 - Site congestion reduced
- Inspections performed at fabrication shop
- Reduced on-site work
- Concrete curing and coating time is drastically reduced



Residual Heat Removal System - Pipe / Valve Module



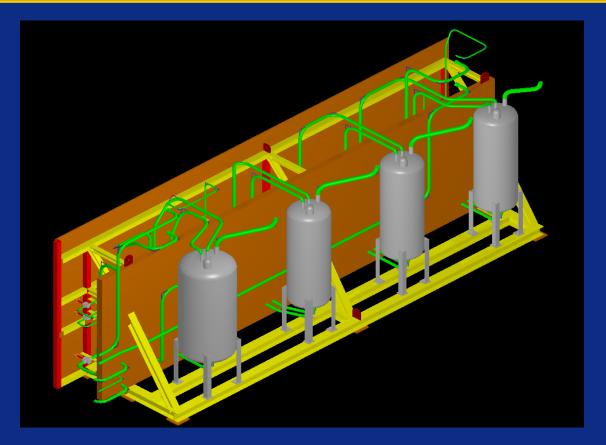
Chemical and Volume Control System Equipment Module





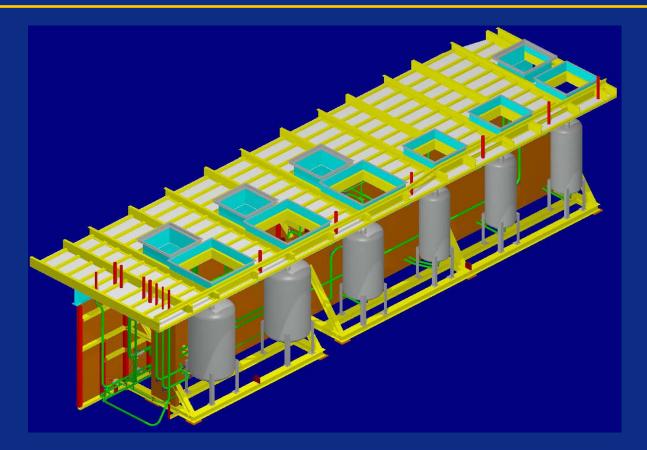
Waste System Demineralizers

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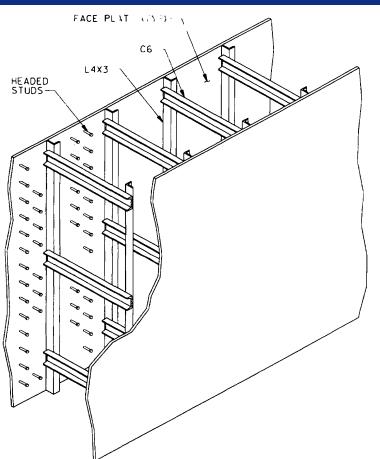
Integration Three Waste Modules



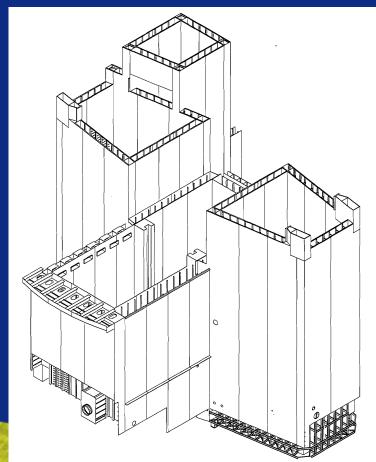


Structural modules

Truss Wall

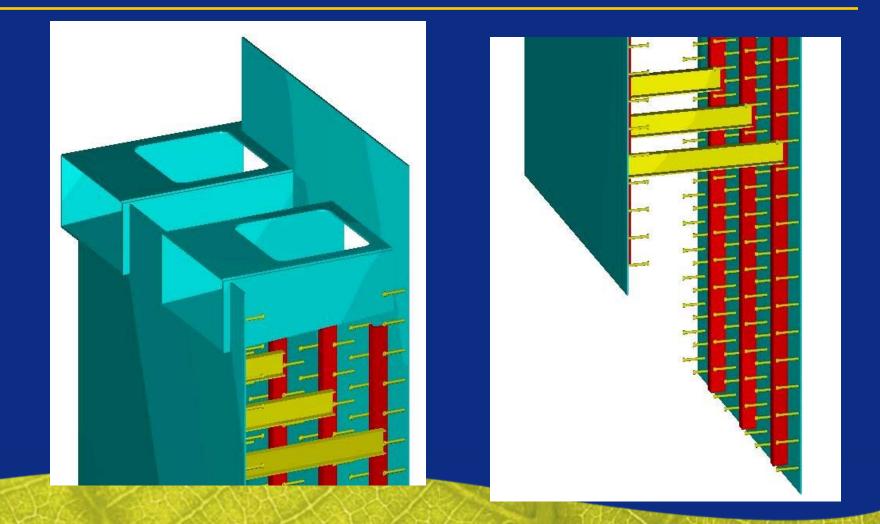


Structural





CA01 Submodule Details





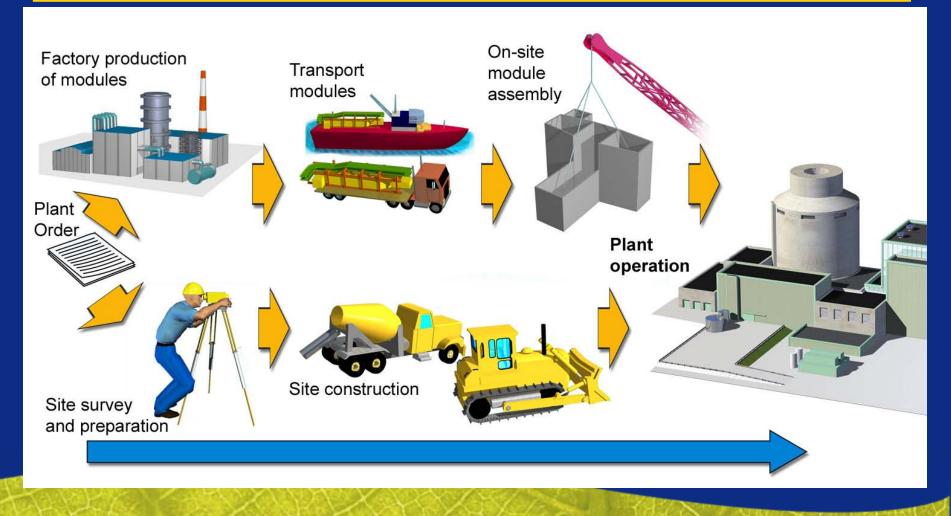
Schedule and Model Integration

Intergraph design review product Primavera Project Planner Continuing interactions with software vendor 3D model boundaries modified to match schedule activities Link by activity Activity construction durations maintained Schedule improvements by logic changes only Visualization capability for changing activity sequences





Modular Construction Allows More To Be Done in Parallel Result: Shorter Construction Schedule

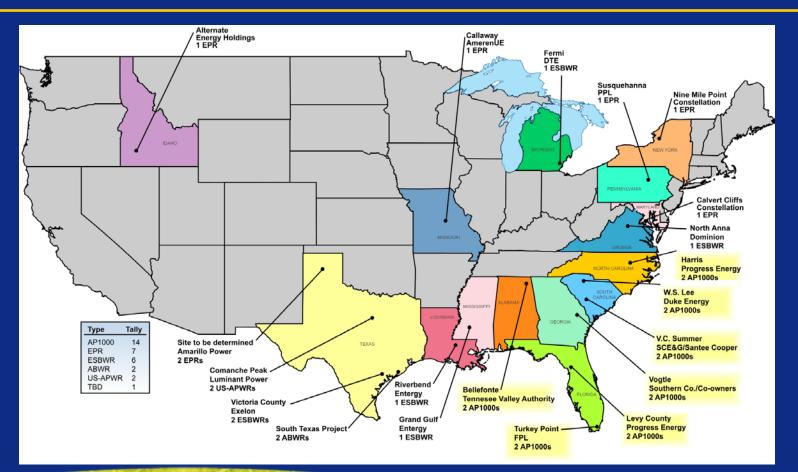




AP 1000: Ten Plants Under Construcion China and USA



The Renaissance Is Here New Reactor License Applications in U.S.



Source: NRC



The Renaissance Is Here <u>AP1000™ Projects in U.S.</u>

14 AP1000 units selected by U.S. utilities to date

6 AP1000 units under contract

SCE&G

VC Summer 2	C.O.D. 4/1/2016
VC Summer 3	C.O.D. 1/1/2019

Southern Co.

Vogtle 3	C.O.D. 4/1/2016
Vogtle 4	C.O.D. 4/1/2017

Progress Energy

Levy County 1	C.O.D. 7/1/2016
Levy County 2	C.O.D. 7/1/2017





AP1000 China Projects

Two units at Haiyang Two units at Sanmen

Construction on Schedule



	Fuel Load	<u>COD</u>
Sanmen #1	May 2013	Nov 2013
Sanmen #2	Mar 2014	Sept 2014
Haiyang #1	Nov 2013	May 2014
Haiyang #2	Sept 2014	Mar 2015



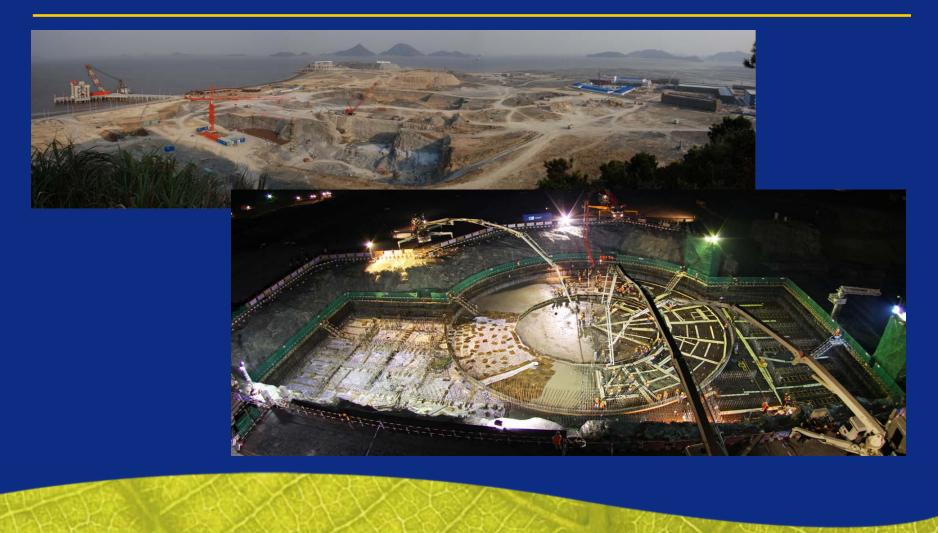
Status of Construction Milestones

- All four NI excavations have been completed
- Sanmen 1 First Concrete Milestone completed on schedule
- Successful setting of CA20
- Auxiliary Building walls started from 66'6" to 82'6"
- Manufacturing of heavy components is on schedule
- Valve development & Demonstration program on track
- Containment Vessel final assembly is occurring

- Reactor Coolant Pump build and test plan is on schedule
- All long lead forgings have been ordered and manufacturing is underway
- Module Factory in China was operational May 31, 2008



The Renaissance Is Here First AP1000[™] Contracts in China





AP1000 - Sanmen First Concrete Pour

March 2009 Placement of the basemat concrete for Sanmen Unit 1 successfully ended March 31 after 46 hours and 58 minutes of continuously pouring 4,982

cubic meters of concrete.









The Renaissance Is Here First AP1000[™] Contracts in China





The Renaissance Is Here First AP1000[™] Contracts in China



U.S. AP100[™] Projects: Project Update Vogtle Units 3 & 4 Project Update

- Project is implemented
 - Westinghouse and Shaw Home Offices are staffed, initial systems and procedures are in place, and implementing Project per schedule
 - Site mobilized and early phases of construction are underway
- PSC certification approved March 17, 2009
- Full Notice to Proceed received from Southern Nuclear Company on March 31, 2009
- 17 of 19 early procurements have been placed (remaining two will be placed by August 2009)



U.S. AP100[™] Projects: Project Update Vogtle Units 3 & 4 Project Update

Construction

- Issued Notice of Commencement
- Continued site mobilization, temporary trailers installed, subcontractors mobilized
- Construction access road completed
- Interim security control established
- Runoff ponds complete, 100-year run-off ditch relocated
- Preparing for plant site excavation



Vogtle 3 &4 March 23 – 30, 2009

U.S. AP100[™] Projects: Project Update VC Summer 2 & 3 Project Update

Activities

- PSC certification of project approved February 11, 2009
- Continuing site preparations including construction access road completion, installation of catch basins, development of lay-down areas, construction city development and rail spur rough grading
- Placed purchase orders for major components including steam generators, reactor vessel, reactor coolant pumps, containment vessel, turbine generator, pressurizer, reactor coolant piping, reactor vessel internals and CRDMs







U.S. AP100[™] Projects: Project Update VC Summer 2 & 3 Project Update

Activities (continued)

- Issued project schedule performance baseline April 1, 2009
- Initiated grading plant access road and the turn lanes at the intersection of Parr Road with State Highway 213 (main access to site)
- Received State Regulatory approval to start clearing and grading the table top in preparation for excavation (Phase 3A permit)
- Upcoming Work
- Commence table top grading in May 2009
- Complete rail spur installation in June 2009

Complete new hire and admin. support buildings in September 2009

Complete Mayo Creek bridge construction Jan. '10





U.S. AP100[™] Projects: Project Update Levy County 1 & 2 Project Update



- "Determination of Need" petition unanimously approved by Florida PSC in July 2008
- Levy County 1 & 2 COLA submitted by Progress Energy to NRC on July 30, 2008
- EPC contract signed on December 31, 2008



U.S. AP100[™] Projects: Project Update Levy County 1 & 2 Project Update

- Consortium home office teams have been assembled and are fully engaged with Progress Energy
- Long-lead procurement of major components commenced in 2008
- Progress Energy notified consortium on April 30, 2009 of partial suspension of work
- On August 13, 2009 Progress Energy received state site certification from the Florida Power Plant Siting Board
- Expect to receive an NRC combined construction permit-operating license in late 2011 or early 2012



AP 1000 The Renaissance Has Started!

THANKS

