

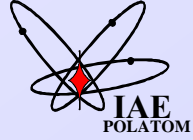
Instytut Energii Atomowej POLATOM

**Institute of Atomic Energy POLATOM
OTWOCK-SWIERK
POLAND**

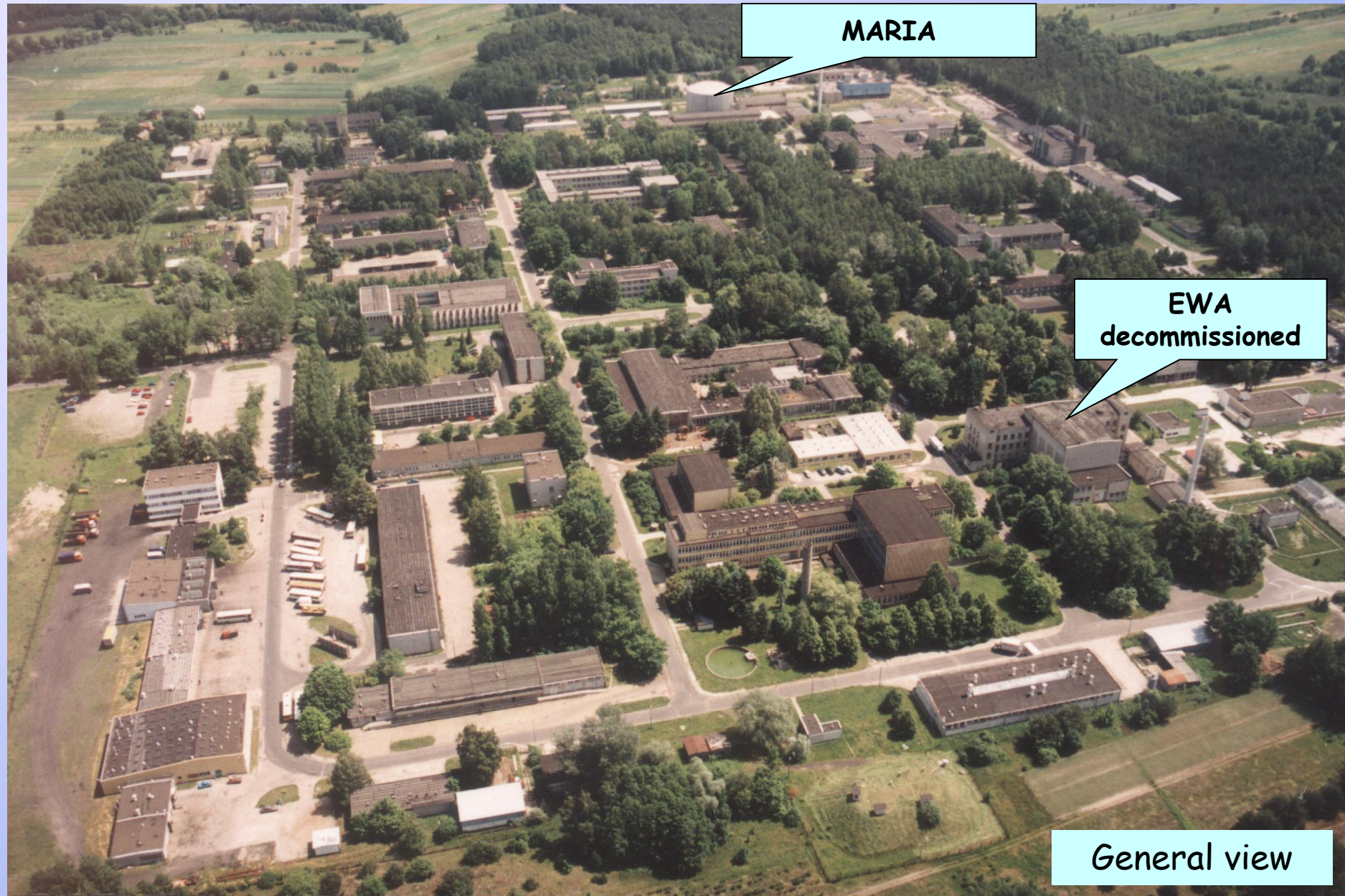
**Irradiations of HEU targets in MARIA RR
for Mo-99 production**

G. Krzysztozek

**IAEA TM on Commercial Products and Services of
Research Reactors
Vienna, 28 June - 2 July 2010**



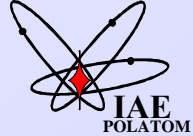
Instytut Energii Atomowej POLATOM



MARIA

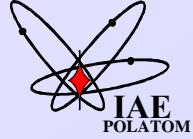
EWA
decommissioned

General view



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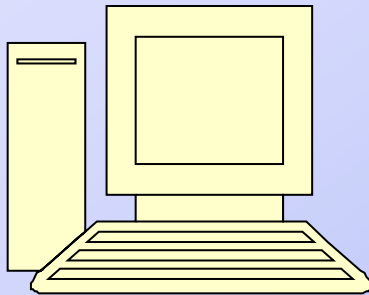


INTRODUCTION

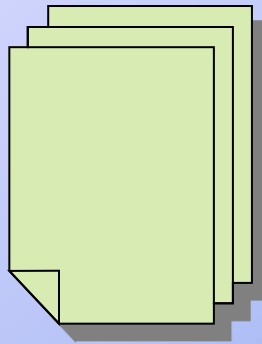
- In May 2009 the NRU reactor (Canada) was shutdown and was planned for scrambling the HFR reactor (Holland),
- In the half of 2009 a decision was taken on cooperating between IAE and COVIDIEN,
- IAE and COVIDIEN initiative cover an irradiation of high-enriched uranium plates in MARIA reactor for production of molybdenum Mo-99 in Petten,
- There was developed the Mo-99 irradiation and transport technology in MARIA reactor facility and then its expedition to the reprocessing factory in Petten (Holland).

INTRODUCTION

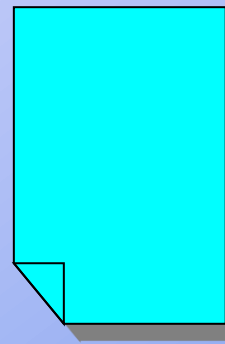
Safety analyses & Design



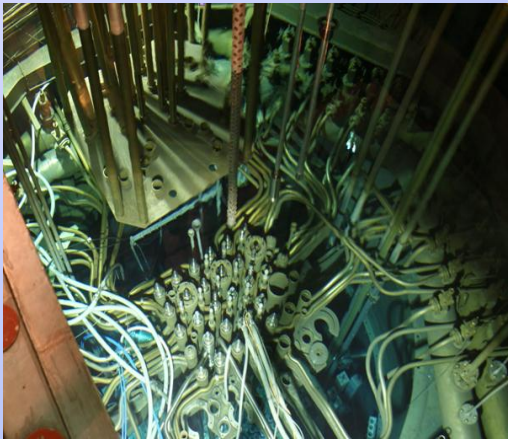
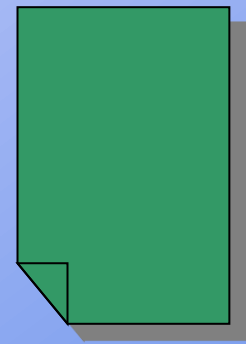
Safety Commission Approval



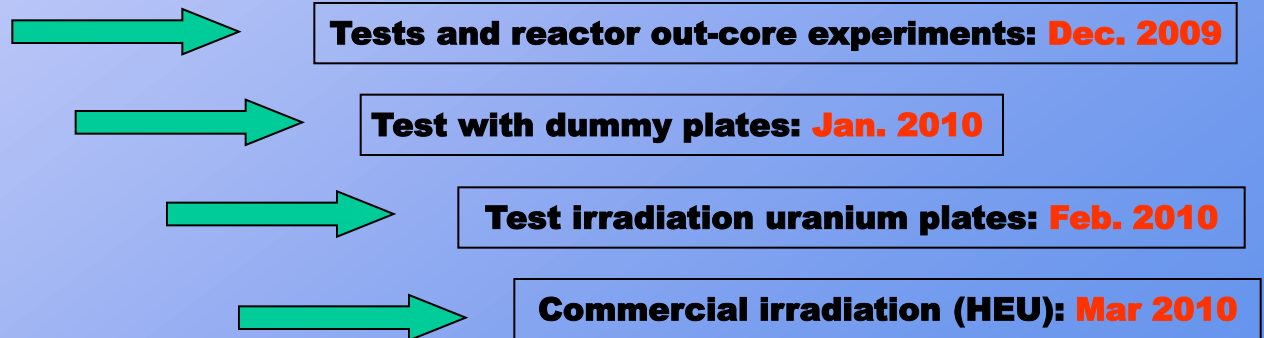
National Regulatory Body Approval

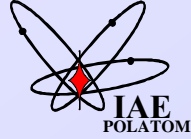


Domestic & Foreign Licenses



**Irradiation facility
manufacture**

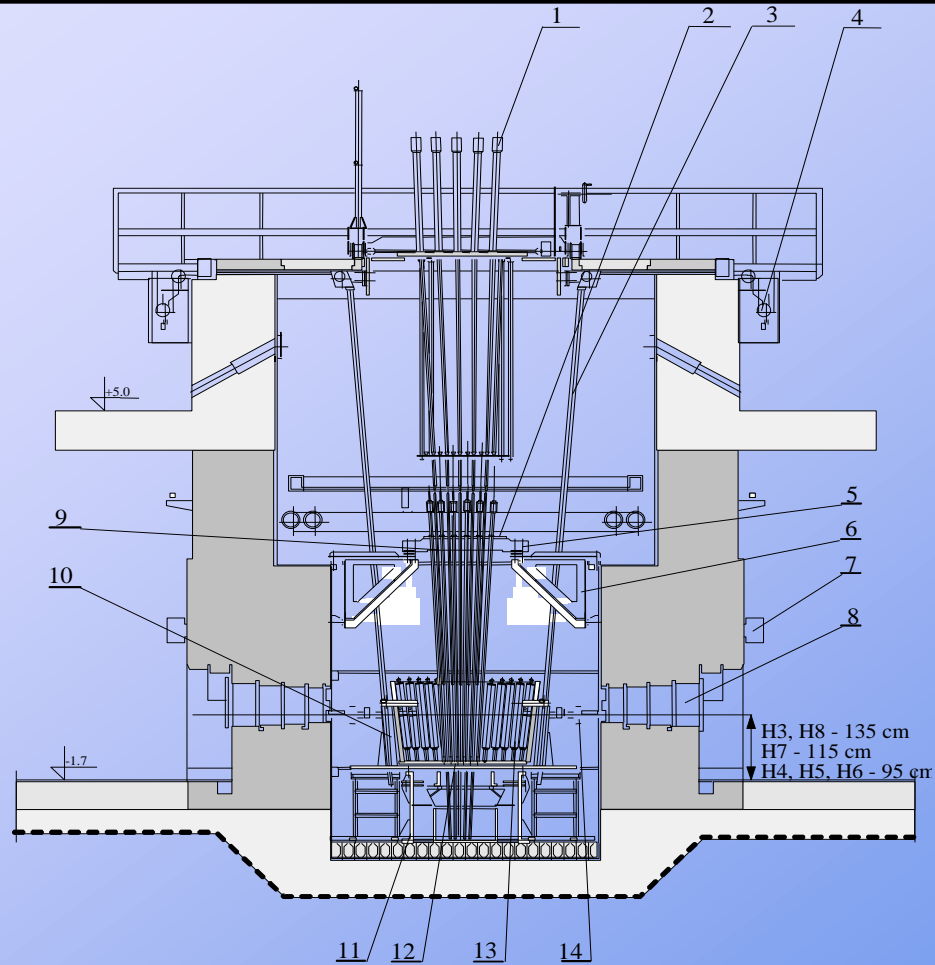




MARIA RESEARCH REACTOR

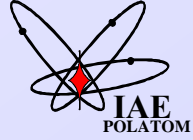
Facility description

- The high flux reactor MARIA is a water and beryllium moderated reactor of 30 MW power level;
- Pool type reactor with pressurized fuel channels containing concentric tube assemblies of fuel elements;
- Fuel channels are situated in matrix containing beryllium blocks surrounded by graphite reflector;
- Main characteristics and data of MARIA reactor:
 - maximum power 30 MW (th)
 - thermal neutron flux density $2,5 \times 10^{14}$ n/cm² s
 - moderator H₂O, beryllium
 - reflector graphite in Al
 - cooling system channel type



- | | |
|---|--------------------------------------|
| 1. control rod drive mechanism | 8. beam tube shutter |
| 2. mounting plate | 9. fuel channel |
| 3. ionization chamber channel | 10. ionization chambers shield |
| 4. ionization chamber drive mechanism | 11. core and support structure |
| 5. fuel and loop channels support plate | 12. core and reflector support plate |
| 6. plate support console | 13. reflector blocks |
| 7. horizontal beam tube shutter drive mechanism | 14. beam tube compensator joint |

FIG. 1. Vertical section of the MARIA reactor.



The main areas of reactor application are as follows:

- production of radioisotopes,
- testing of fuel and structural materials for nuclear power engineering,
- research in neutron and condensed matter physics,
- neutron radiography,
- neutron activation analysis,
- neutron transmutation doping.

FUEL ELEMENT - MR

- Material UO_2 -Al. Alloy clad in aluminium
- Enrichment 80% / 36% U-235 (from 1999)
- Shape concentric tubes
- Dimensions 1000 mm height / 79 mm diameter
- Cooling under pressure flow
- Power limited to 1.8 MW
- 2 Lead Test Assemblies MC-LEU (19,7% U-235) are irradiated for qualification

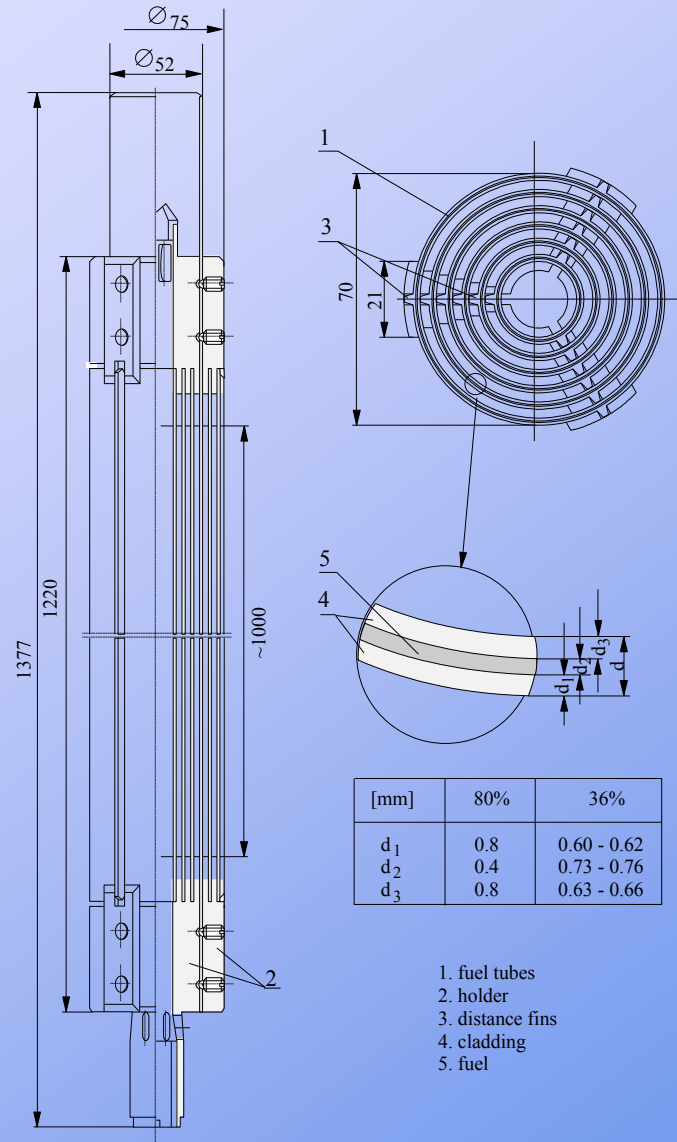
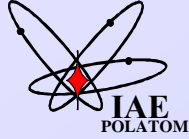


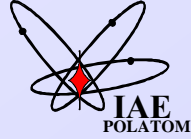
FIG. 2. MR - Fuel element



**DEVELOPING THE URANIUM TARGETS
IRRADIATION TECHNOLOGY, SAFETY ANALYSIS,
MEASUREMENTS AND TESTS**

**Technology for irradiation and handling of uranium plates
comprise of:**

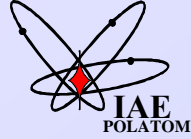
- Irradiation of plates and initial cooling in the irradiation channel
- Calorimetric measurement of heat generation in the capsule with plates
- Transport of plates into the hot cell
- Handling operations in the hot cell
- Loading of plates into the transport cask MARIANNE



DEVELOPING THE URANIUM TARGETS IRRADIATION TECHNOLOGY, SAFETY ANALYSIS, MEASUREMENTS AND TESTS

Calculations and safety analyses at steady states are as follows:

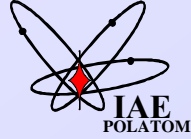
- Calculations of molybdenum activity
- Neutronic calculations
- Thermal-hydraulic calculations at steady states
- Activity of fission products and thermal power of the uranium plate batch
- Cooling of uranium plates in the capsule for irradiation during natural convection in the air
- Shielding calculations and an assessment of radiological hazard for personnel pending reloading – transport operations



DEVELOPING THE URANIUM TARGETS IRRADIATION TECHNOLOGY, SAFETY ANALYSIS, MEASUREMENTS AND TESTS

Program of examinations and installation tests consist of:

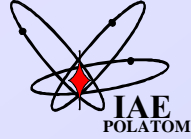
- Hydraulic measurement of channel for irradiation of capsules containing the mock-ups of plates
- Cold trials of reloading and transport operations with a bath of dummy plates
- Calibration measurements of calorimeter for measuring of thermal power of 4 plate batch
- Measurement of axial distributions of the neutron flux density in the capsule containing dummy plates
- Measurements of the heat balance in molybdenum installation with the dummy plates
- Test irradiation of uranium plates and their dispatching
- Measurements of temperatures of uranium plates in the air



DEVELOPING THE URANIUM TARGETS IRRADIATION TECHNOLOGY, SAFETY ANALYSIS, MEASUREMENTS AND TESTS

Technical Assumptions for developing the uranium targets technology:

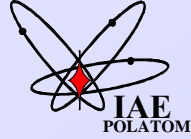
- Irradiation is held in containers, containing 4 plates each, loaded into the molybdenum channel.
- Irradiation of the containers is held in installations which are converted fuel channels of the MARIA reactor.
- Loading and discharge of the containers with plates from the installation is possible without the necessity of the evacuation of the irradiation channel from the reactor core.
- The nominal flow of coolant is maintained in the irradiation installation.



DEVELOPING THE URANIUM TARGETS IRRADIATION TECHNOLOGY, SAFETY ANALYSIS, MEASUREMENTS AND TESTS

Technical Assumptions for developing the uranium targets technology:

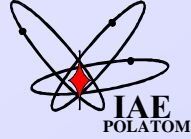
- The cooling is ensured by the circuit of cooling fuel channels.
- Opening of the molybdenum channel and the evacuation of the containers with uranium plates are held of not earlier than 10 hours after the reactor shutdown.
- Handling operations in the hot cell are conducted in the air.
- Cooling plates with the natural convection in the air is less efficient than cooling convection in water.



DEVELOPING THE URANIUM TARGETS IRRADIATION TECHNOLOGY, SAFETY ANALYSIS, MEASUREMENTS AND TESTS

Technical Assumptions for developing the uranium targets technology:

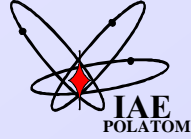
- The recipient of uranium plates determines two thermal limits for the set of 8 uranium plates (residual power 548 W and 450 W).
- The procedure of the uranium plates dispatch includes the possibility of conducting calorimetric measurements of the residual heat generated in a single container with plates.
- The measurements of plate's temperature were conducted in hot cell during test irradiation.
- These measurements showed that temperatures of uranium plates in the air were below 200 °C.



**DEVELOPING THE URANIUM TARGETS
IRRADIATION TECHNOLOGY, SAFETY ANALYSIS,
MEASUREMENTS AND TESTS**

**Technical Assumptions for developing the uranium targets
technology:**

- The total activity of fission products in uranium plates during transport operations in the hot cell is ca. 100 kCi.
- Test measurements showed that the shielding of the hot cell is sufficient for safe performing handling operation of irradiated uranium plates.
- After the process of irradiation and cooling plates has ended the plates are loaded into a special shielding container MARIANNE



MO-99 PRODUCTION IN THE MARIA REACTOR – CURRENT STATE

- Between 8th ÷ 14th February the test irradiation of 8 plates has been conducted.
- In the period from 11th March to 2nd June 2010 eight irradiation cycles in molybdenum channels in the MARIA reactor were conducted.
- In all cycles 12 sets of uranium plates (8 plates in each) were irradiated.
- Irradiations were conducted in three different locations of molybdenum channels (f-7, h-7 and i-6) and different configurations of the core, introduced in *FIG.3*.

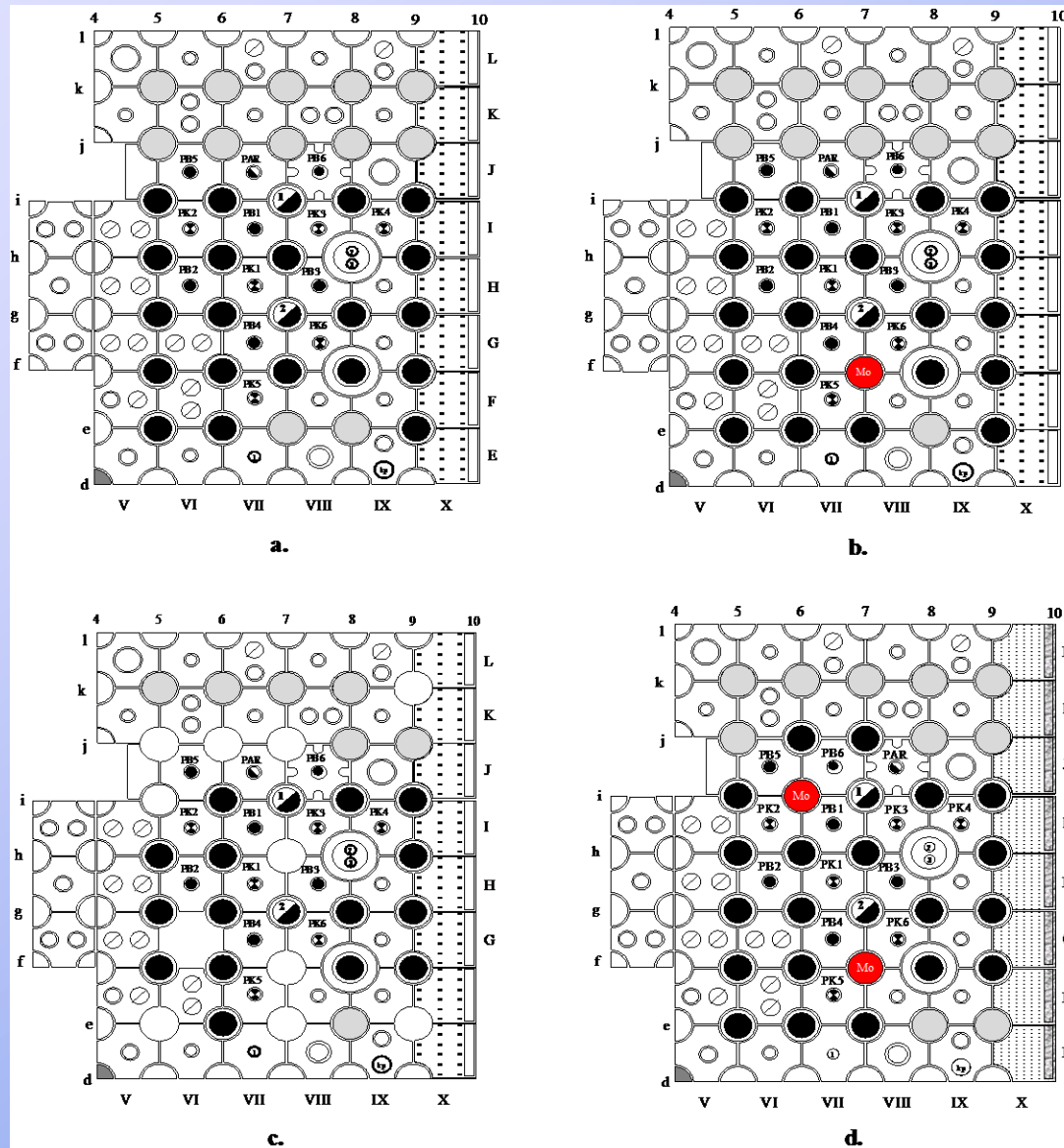
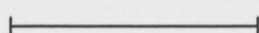





FIG. 3. Configurations of the MARIA reactor core with molybdenum channels.

Schedule of reactor MARIA operation in 2010

Date of actualization 04.03.2010

	Mo	Tu	We	Th	Fr	Sa	Su	Mo	Tu	We	Th	Fr	Sa	Su	Mo	Tu	We	Th	Fr	Sa	Su	Mo	Tu	We	Th	Fr	Sa	Su	Mo	Tu	We	Th	Fr	Sa	Su	Mo	Tu	
January					1 (1)	2	3	4 (2)	5	6	7	8	9	10	11 (3)	12	13	14	15	16	17	18 (4)	19	20	21	22	23	24	25 (5)	26	27	28	29	30	31			
February	1 (6)	2	3	4	5	6	7	8 (7)	9	10	11	12	13	14	15 (8)	16	17	18	19	20	21	22 (9)	23	24	25	26	27	28										
March	1 (10)	2	3	4	5	6	7	8 (11)	9	10	11	12	13 (12)	14	15 (12)	16	17	18	19	20	21	22	23 (13)	24	25	26	27	28	29	30	31 (14)							
April			1 (14)	2	3	4	5	6	7	8	9	10	11	12 (16)	13	14	15	16	17	18	19 (17)	20	21	22	23	24	25	26 (18)	27	28	29	30						
May				1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31 (23)				
June	1 (23)	2	3	4	5	6	7	8	9	10	11	12	13	14 (25)	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30								
July		1 (27)	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31						
August				1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31 (36)				
September	1 (36)	2	3	4	5	6 (37)	7	8	9	10	11	12	13 (38)	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30								
October			1 (40)	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31					
November	1 (45)	2	3	4	5	6	7	8 (46)	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30								
December		1 (49)	2	3	4	5	6 (50)	7	8	9	10	11	12	13 (51)	14	15	16	17	18	19	20 (52)	21	22	23	24	25	26	27	28	29	30	31						

 Operation
 Maintenance
 Nr of week
 Nr cycle

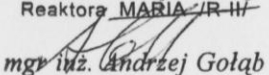
KIEROWNIK
 Zakładu Eksploatacji
 Reaktora MARIA-IR III

 mgr inż. Andrzej Gołąb

FIG. 4. Schedule of reactor MARIA operation in 2010.

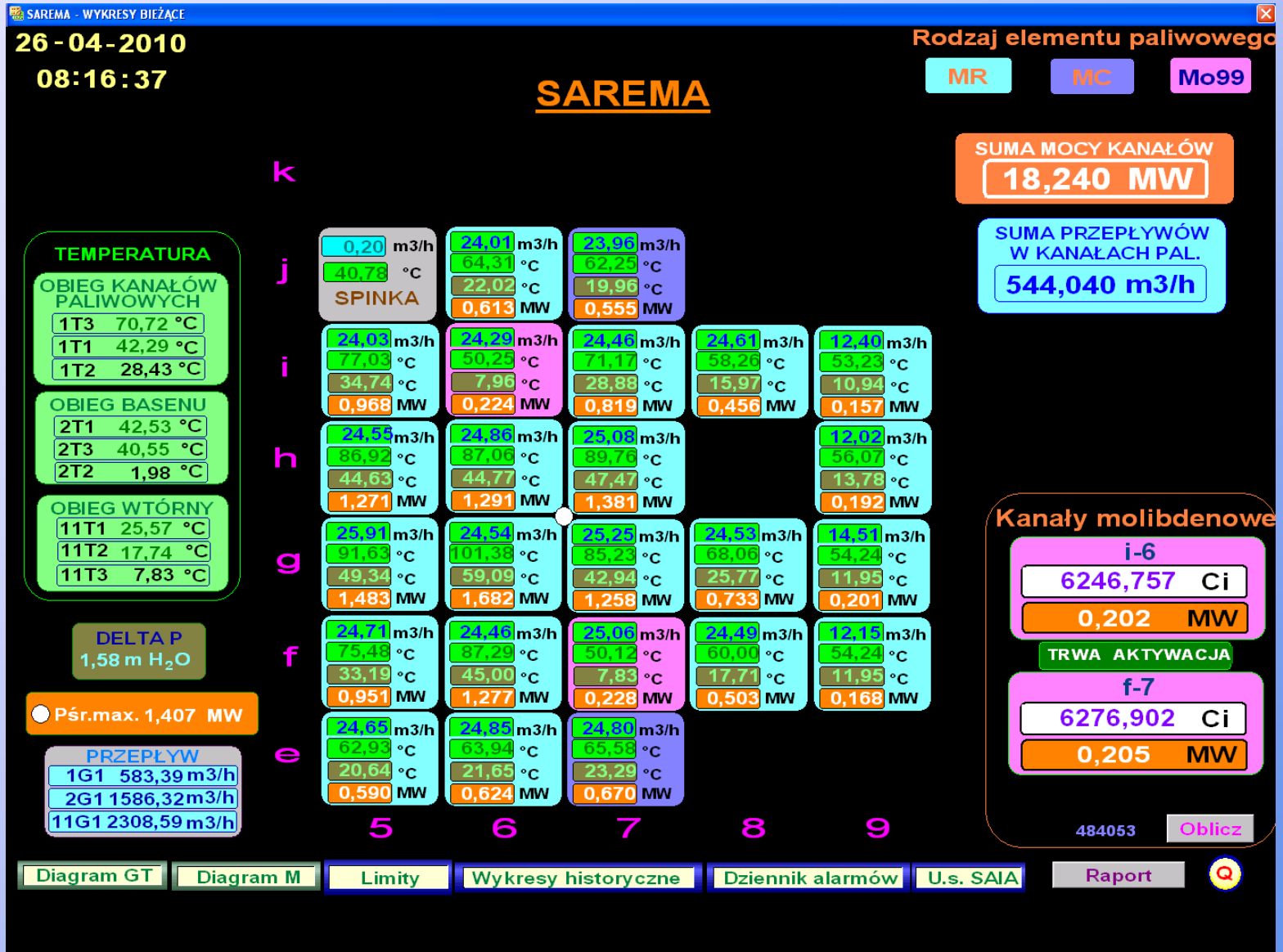
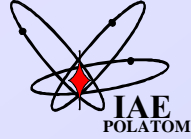


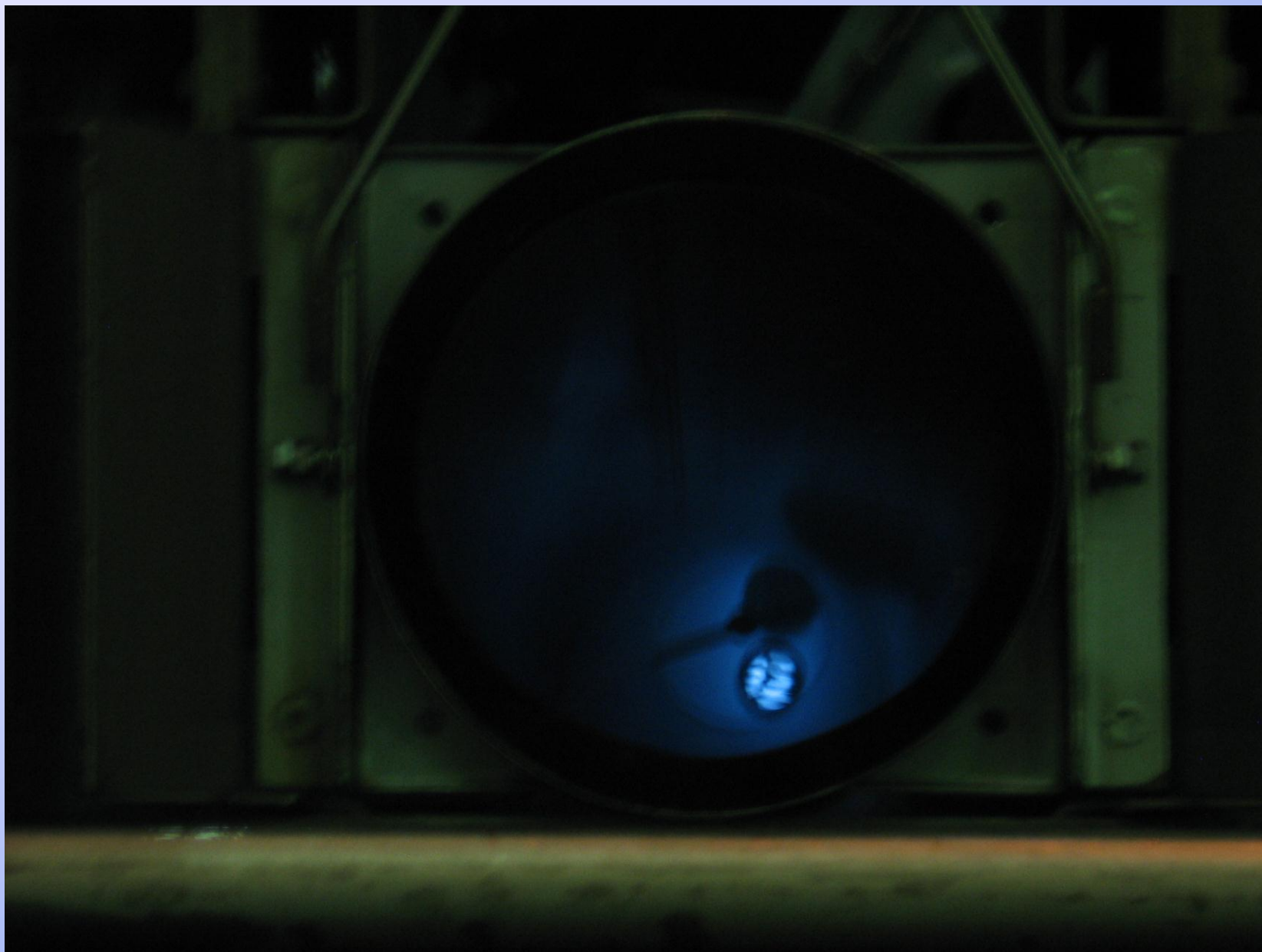
FIG. 5. Diagram of MARIA RR core.



HANDLING OF IRRADIATED TARGET IN MARIA REACTOR BUILDING

- After reactor shut down and cooling time minimum 12 hours the irradiation holders are unloading from the channels and handling to the dismantling cell.
- Calorimetric measurements conducted directly before the dispatch of plates showed residual powers in the range of $320 \div 410$ W, that is below limit 450 W of power.
- Measurements of the temperature of plates in the container in conditions of the natural convection in the air were made after test irradiation has ended.
- The measurement was performed directly before plates were loaded into the shielding container MARIANNE.
- The temperature of plates didn't achieve the value of 200 °C.

FIG. 6. Handling of irradiated target in MARIA reactor building.





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Świerk, 03 June 2010

CERTIFICATE OF RADIOACTIVE SOURCE no. 15/10

1. Preparation: 8 uranium targets / irradiation container: 29/10 and 30/10
2. Irradiation Rig: channel i-6
3. Batch: 2010.13/311205

Position holder	Target number	Position holder	Target number
LOWER	NRGA3911	UPPER	NRGA3915
	NRGA3912		NRGA3916
	NRGA3913		NRGA3917
	NRGA3914		NRGA3918

4. Radionuclide: Mo-99
5. Mo-99 activity/batch: 7773 Ci on EOI *91a - Mo*
6. Residual power: < 410 W *91u*
7. Container type: MARIANNE no. 01 *10.06.07*
8. Dose rate equivalent on the outer surface of container: *0.02* mSv/h
9. Time of irradiation:

Start of irradiation:	27 May 2010 – at 18:20
Stop of irradiation:	02 June 2010 – at 14:45
11. Total: 140h 25'

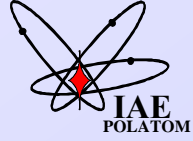
Dosimetry
DOZYMETRISTA
Reaktor MARIANNE
Janusz Suchocki

Laboratory
**KOORDYNATOR PROGRAMÓW
NAPROMIENIOWANIA W REAKTORZE
MARIANNE**
mgr inż. Janusz Jaroszewicz



CONCLUSION

- The realization of the molybdenum program confirmed the correctness
 - of the irradiation technology,
 - handling operations in the reactor pools and in hot cell,
 - loading operation into the transport container MARIANNE.
- Experience acquired made it possible to implement additional technical and organizational solutions.
- The achieved very good results of production is an important step in increasing of commercial products and services of MARIA research reactor.



THANK YOU