



# Approaches developed to build TSO capabilities in Morocco and to assess the safety of experiments in its TRIGA Research Reactor

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# Outline

- Presentation of TRIGA Mark II RR
- Safety organization at CNESTEN
- Categorization and review of experiments
- Practical implementation of the approach developed for the production of I-131
- Building TSO capabilities using international cooperation
- Conclusion

# Presentation of TRIGA Mark II Research Reactor

# MAAMORA NUCLEAR CENTER



# Presentation of CENM TRIGA Mark II RR

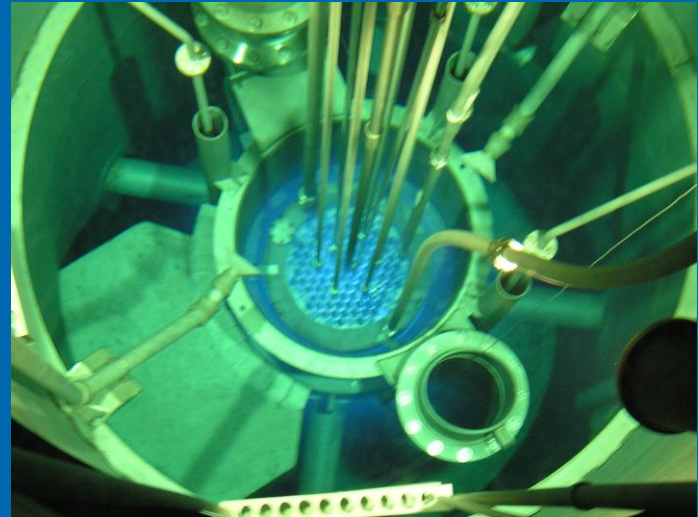
- Design Standard Mk II
- Power 2 MW (t)
- Cooling Natural Convection
- Fuel U-ZrH



# Presentation of CENM TRIGA Mark II RR

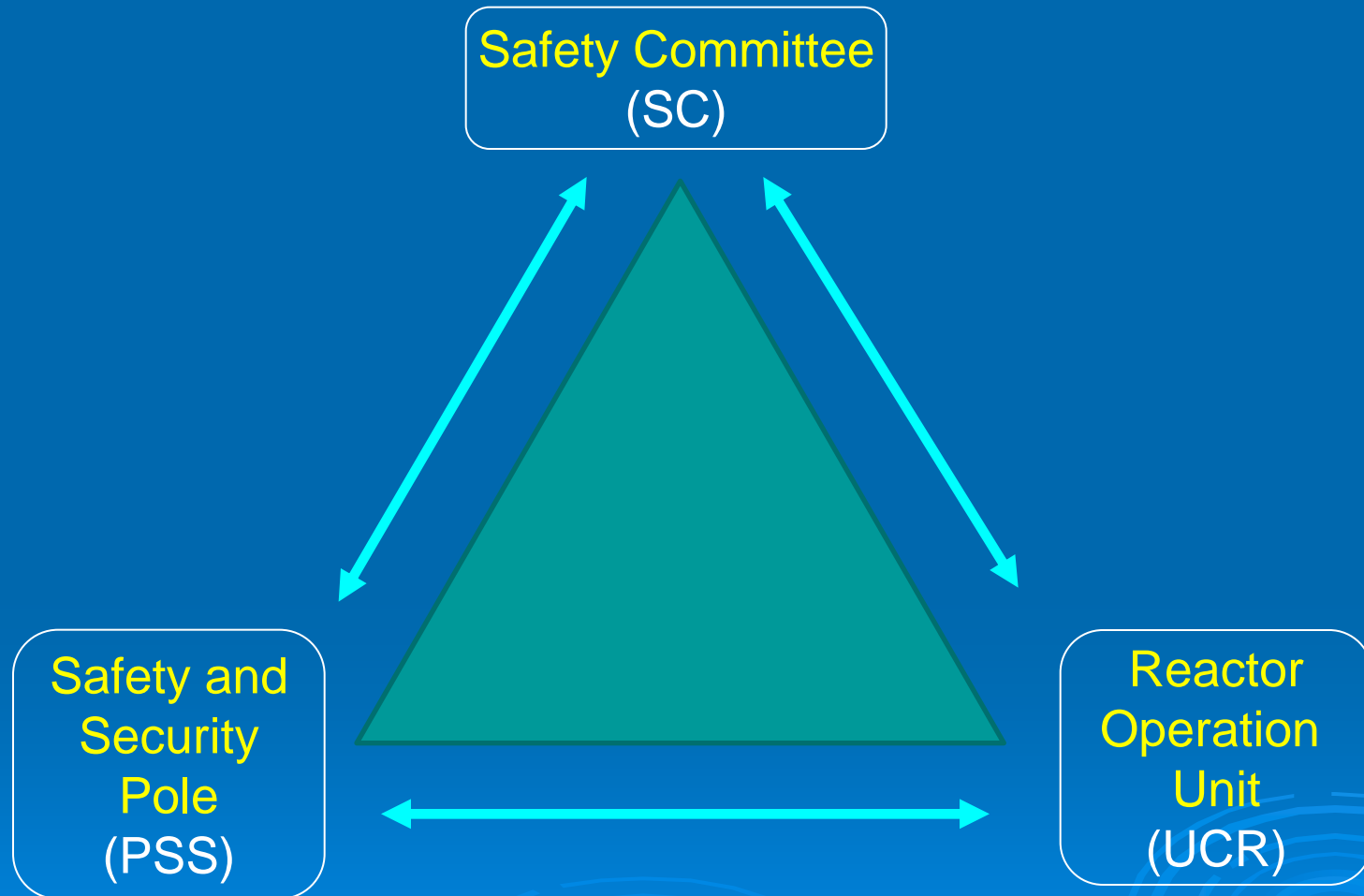
## ➤ Reactor Features

- Graphite Reflector
- Beam Tubes (4)
- Thermal Column
- Pneumatic Transfer System
- Rotary Specimen Rack
- In-pool fuel storage racks



# Safety organization at CNESTEN

# Safety Organization at CNESTEN





# Fields of work of Safety and Security Pole (PSS)

- Nuclear safety;
- Radiological and industrial safety;
- Dosimetry, Calibration, Monitoring network
- Environmental protection;
- Physical protection;
- Emergency preparedness and response;
- Safeguards.



# Safety Committee (SC) : Aspects reviewed

- Commissioning Program (CP) and its results,
- Treatment of abnormal events,
- **Proposed experiments,**
- Modifications having an impact on safety level,
- Rules, procedures and instructions of the reactor operation,

# Safety Committee (SC) : Aspects reviewed (Con't)

- Results of safety audits relative to the operation of reactor,
- Violations of the OLCs, the license and procedures that are significant to safety,
- Events that are required to be reported to the Safety Authority,
- Safety documents.

# Categorization and review of experiments

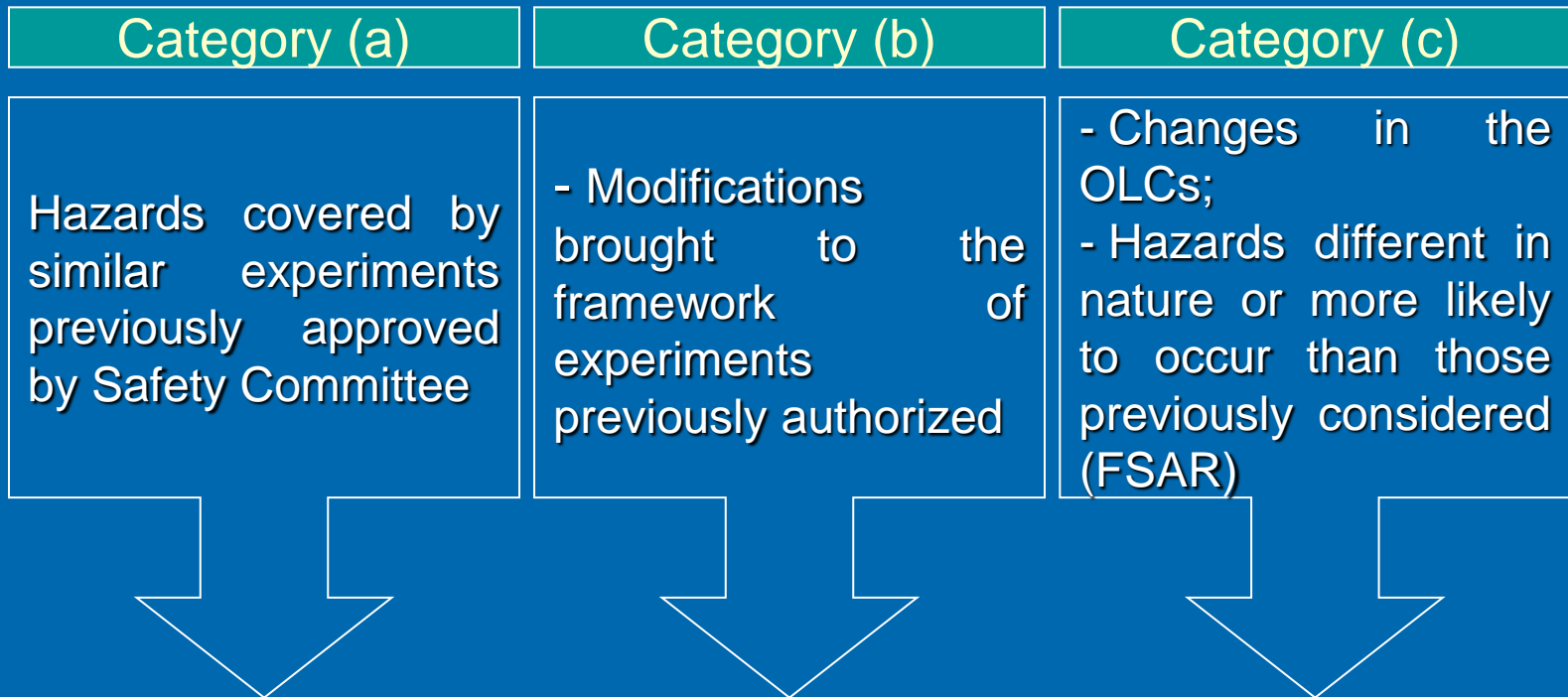
# Establishment of review and approval process

- Establishment by CNESTEN of a draft approach of categorization and review of experiments in compliance with the IAEA draft safety guide DS397 “Safety in the Use and Modification of Research Reactors”
- Review of this approach during an IRSN Expertise Mission (December 2009)
- Approval by CNESTEN of the above mentioned approach

# Framework of experiment safety

- National regulation and licenses;
- Final Safety Analysis Report (FSAR), including OLCs;
- IAEA safety Standards:
  - Code NS-R-4 “Safety requirements for RR”;
  - Guide 35-G2 “Safety in the utilization and modification of RR”;
- International practices (USA, France)

# Experiment classification and review



UCR (Reactor Operation Unit)	Approval	Preparation of Safety doc.	Preparation of Safety doc.
PSS (Safety & Security Pole)		Independent Analysis	Independent Analysis
Safety Committee		Review and approval	Review
Safety Authority			Review and approval



# Safety document

- Purpose and justification for the experiment;
- Experimental devices;
- Characterization of target to be irradiated (confinement) and of reactor operation;
- Impact of experiment on the reactor safety :
  - Reactivity;
  - Thermalhydraulic;
  - Chemical hazards;
- Integrity of confinement;

# Safety document (con't)

- Assessment of doses to reactor staff and presentation of radiation protection measures;
- Calculation of radiological consequences in accident conditions,
- Assessment of compliance to licensing conditions and FSAR;
- Commissioning process;
- Operational procedures;

# Safety document (con't)

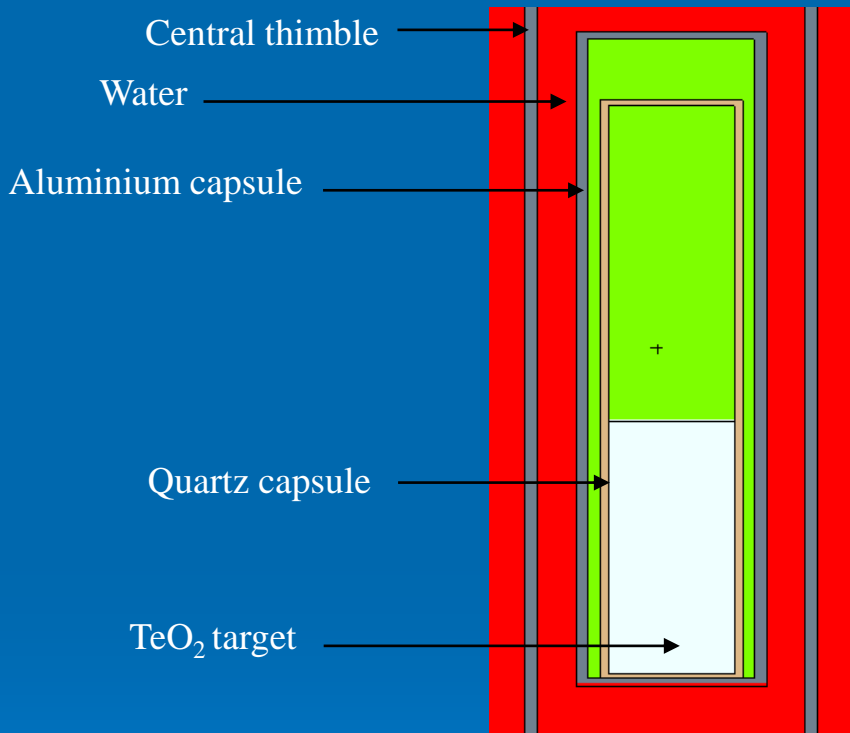
- Radioactive waste and effluents generated;
- Special requirements for the training and, if necessary, relicensing of reactor operators.

# Practical implementation of approach developed for the production of I-131

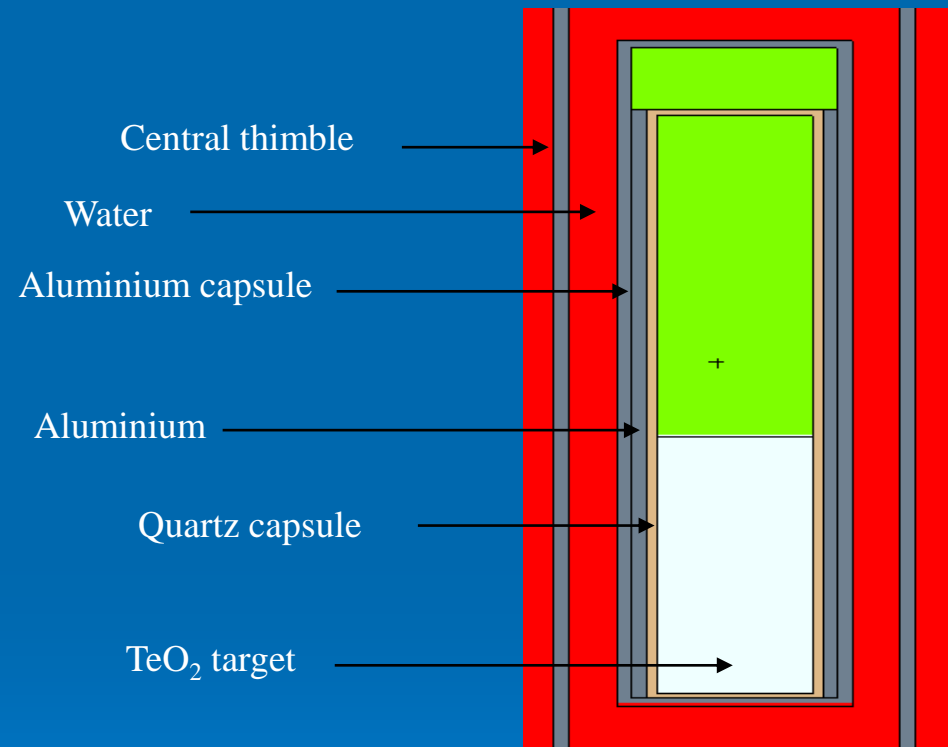
# Review of I-131 production experiment

- Preparation and submission of Safety Document of the experiment by the Reactor Operation Unit
- An independent analysis and a safety assessment were carried out by the Safety and Security Pole (PSS)
- Organization of an IRSN expertise mission to review the safety assessment made by PSS
- Identification of specific safety issues which deserved to be studied in detail, in accordance with the experiments safety assessment approach
- Approval route of this experiment (Safety Committee) :
  - Validation of encapsulation and extraction process
  - Approval to conduct the experiment following a graded approach

# Encapsulation : 2 Configurations

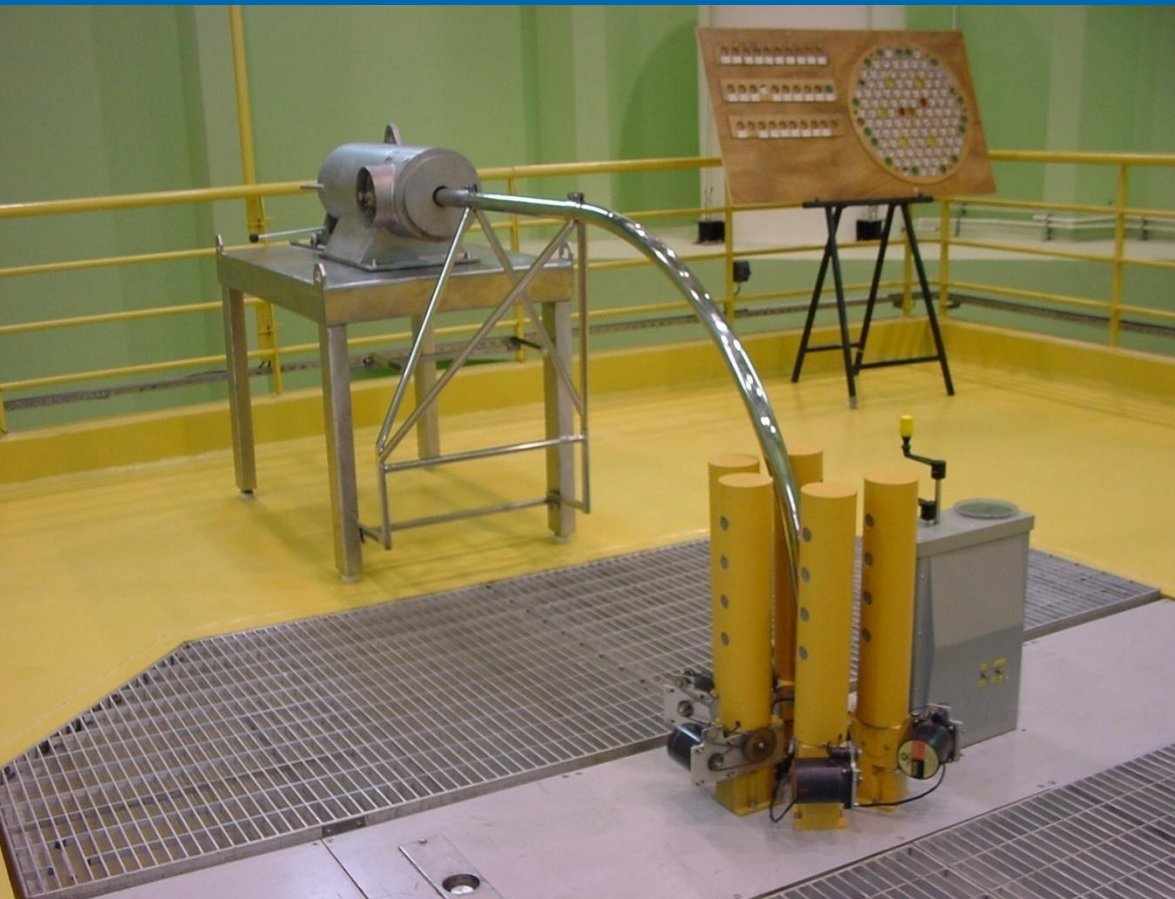


Configuration 1



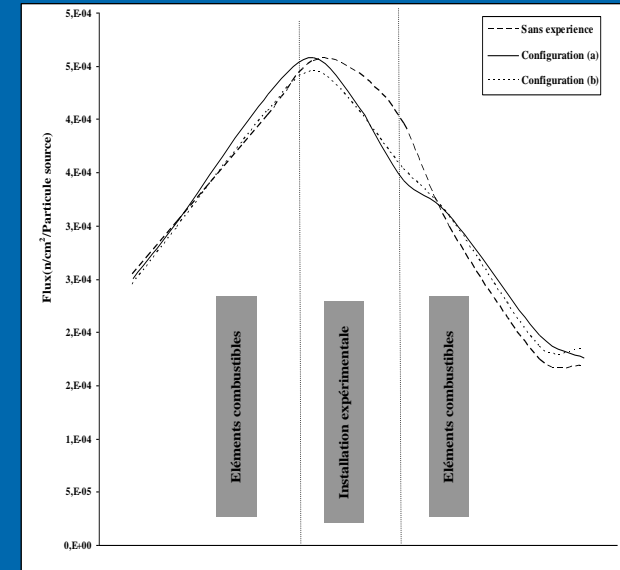
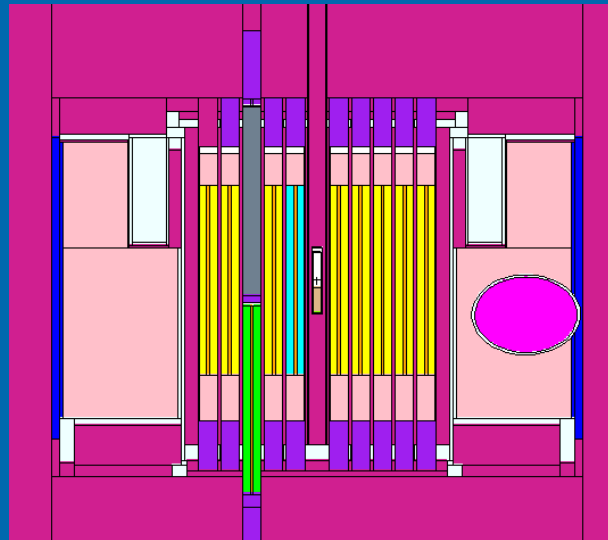
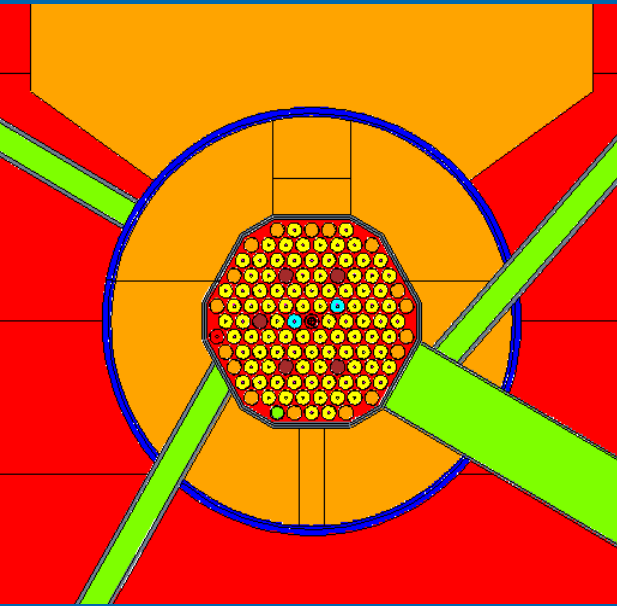
Configuration 2

# Extraction

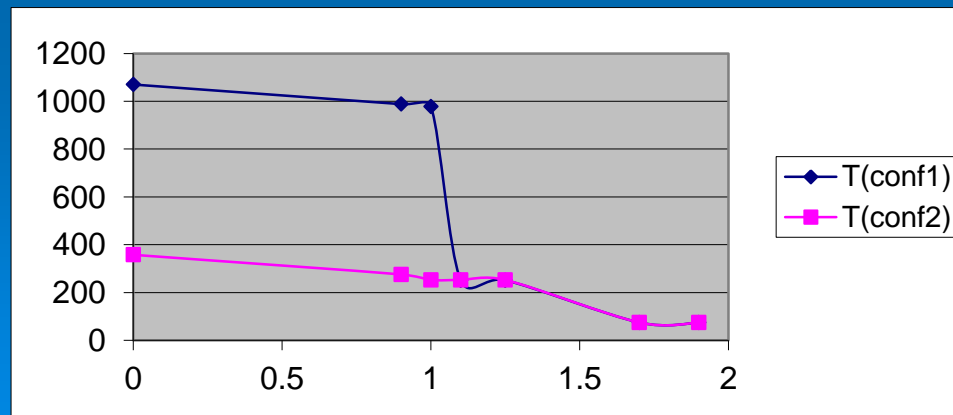




# Independent Safety Analysis (PSS)



Neutron calculation



Thermal calculation

# Independent safety analysis (PSS)

## Radiological impact calculation for accidents

- **IAEA CRP** «Modeling and Analysis of Radionuclides Transport and Source Term Evaluation within Containment / Confinement and Release to the Environment for Research Reactors ».
- **IAEA Safety Report n° 53** « Derivation of the Source Term and Analysis of the Radiological Consequences of Research Reactor Accidents » ;

# Independent safety analysis

## Radiological impact calculation for accidents

- RSAC-6 code (Radiological Safety Analysis Computer program) ;
- Code HOTSPOT (version 2.07) ;
- Microshield Code.



Ref. : Hotspot user guide, March 2010

# Building TSO capabilities using international cooperation

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- A Nuclear and radiological Safety and Security law will be soon adopted by Morocco,
- This law foresees the creation of a new unique and independent Regulatory Body which will need to rely on the support of CNESTEN as TSO for safety assessment and review,
- In this perspective, CNESTEN is reinforcing its human resources and developing skills in framework of international cooperation, mainly with the IRSN, IAEA, European Commission and USA.

# Building TSO capabilities using international cooperation

- Several countries have acquired a huge experience on nuclear plants:
  - The organization of international cooperation for countries willing to launch into nuclear technology is a good mean to update with the state of the art of the “safety culture”.
  - This approach leads to a standardization of safety practices to a high level.
- The CNESTEN acquired an experience in the area of safety through:
  - the follow-up for the TRIGA reactor,
  - International cooperation

# Conclusions



# Conclusions

- CNESTEN has developed the infrastructure needed and capabilities necessary to ensure an independent follow-up on safety level of TRIGA Reactor :
  - The set up of an effective safety committee with required skills,
  - The adoption of an approach regarding the safety review process of experiments constitute important steps to fulfill the national regulations and the IAEA safety standards.
  - The implementation of best practices.
  - The organization at the critical steps of independent safety reviews in framework of international cooperation (IAEA, IRSN),

THANK YOU FOR YOUR ATTENTION